A scenic view of a town, likely in the Mediterranean region, featuring a prominent church spire and a large palm tree in the foreground. The town is built on a hillside, and the background shows distant mountains under a clear blue sky. The foreground includes a green lawn and several palm trees.

Neutron beams for nuclear data measurements

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Germany

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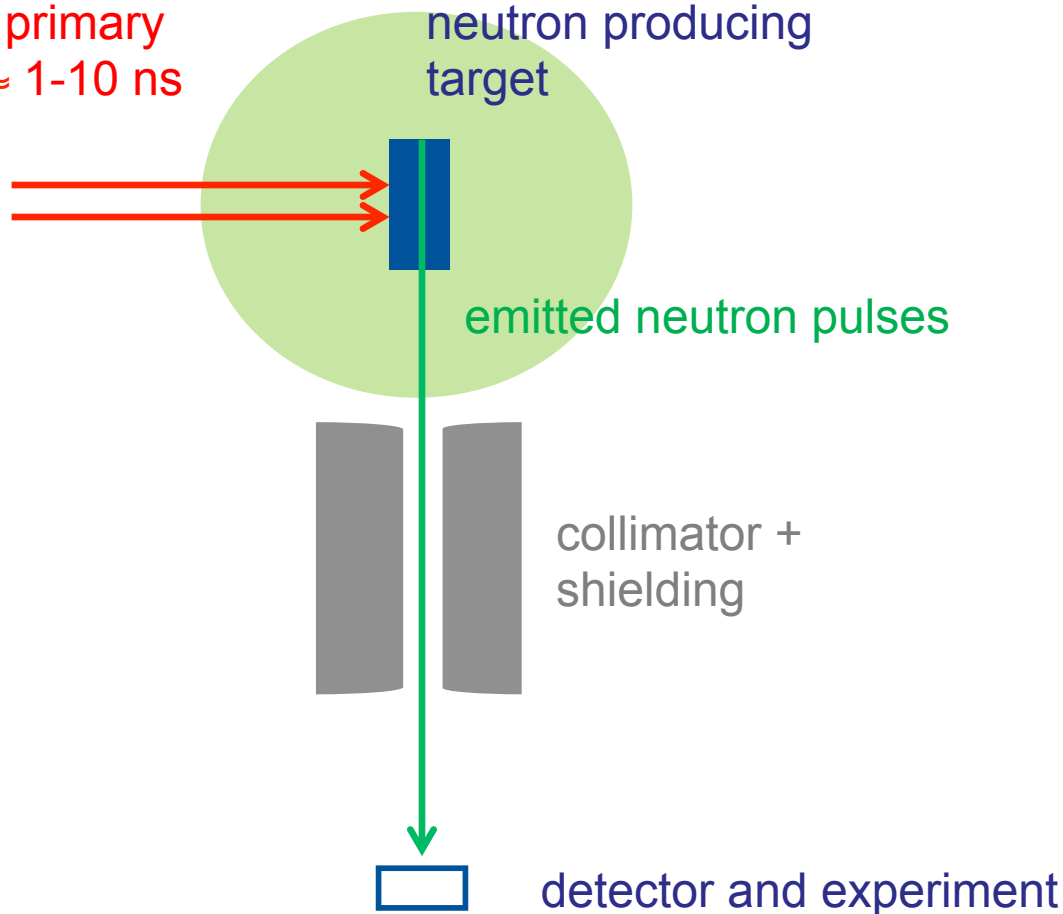
- Introduction of the speaker

- Neutron sources (Terrestrial and radioisotope sources)
- Neutron producing nuclear reactions: Monoenergetic neutrons
- Kinematics e.g. ${}^7\text{Li}(p,n){}^7\text{Be}$
- Neutron reference fields: „Big Four“ reactions for neutron production
- Neutron generators: $\text{D}(d,n){}^3\text{He}$, $\text{T}(d,n){}^4\text{He}$ reactions

- Time-of-flight method and neutron sources
- Spallation neutron sources using light-ion accelerators
- Slowing down of neutrons to produce eV-keV neutrons
- Photoneutron sources using electron accelerators nELBE, Gelina
- NFS at GANIL

Schematic time of flight measurement

pulsed primary beam $\approx 1-10$ ns



primary beams:
light charged particles with
>100 MeV energy
→ spallation neutron source

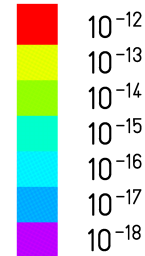
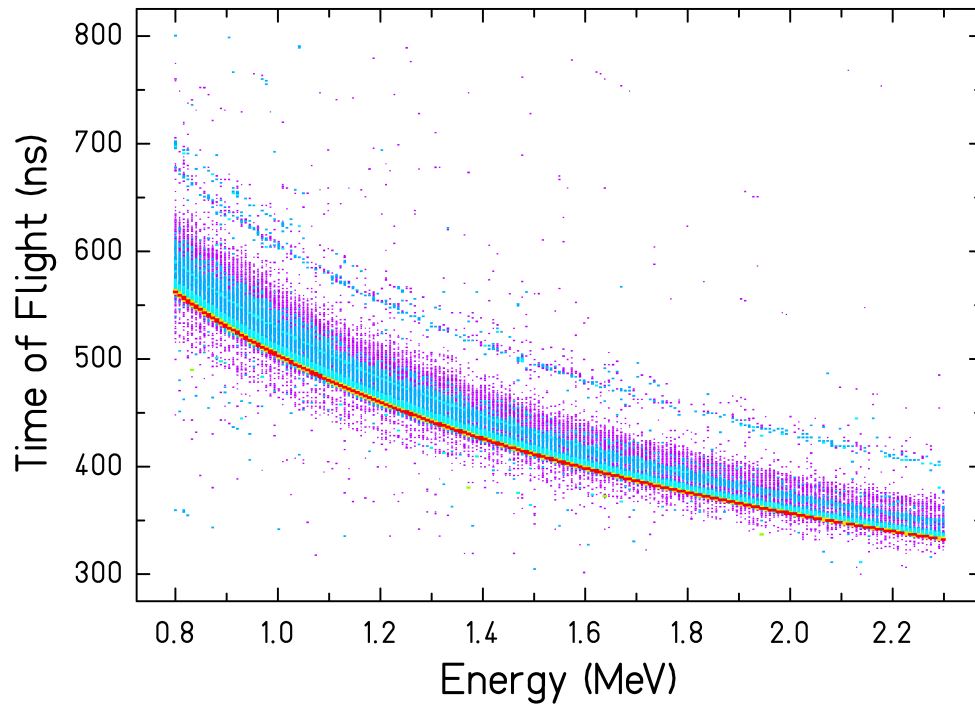
electrons 10 – 150 MeV
→ photoneutron source

Two body kinematics →
quasimonochromatic neutron
sources e.g. ${}^7\text{Li}(p,n)$

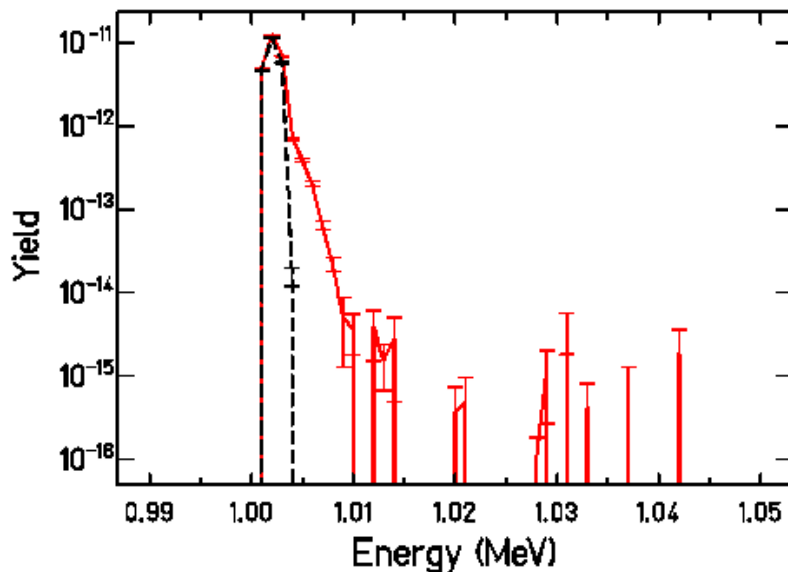
1-2-3 of time of flight:

- Measurement of time-of-flight t and flight path l
 1. $v=l/t$
 2. $\gamma=1/\sqrt{1-(v/c)^2}$
 3. $E=mc^2(\gamma-1)$ (E is the neutron kinetic energy)
- Energy resolution
 1. $\Delta E/E = (\gamma+1)\gamma\Delta v/v$
 2. $\Delta v/v = \sqrt{(\Delta t/t)^2 + (\Delta l/l)^2}$
- accelerator pulse length, time resolution of detectors, neutron transport in the neutron producing target and detector or sample [Schillebeeckx et al. NDS 113 \(2012\) 3054](#)

Time-of-flight to Energy correlation

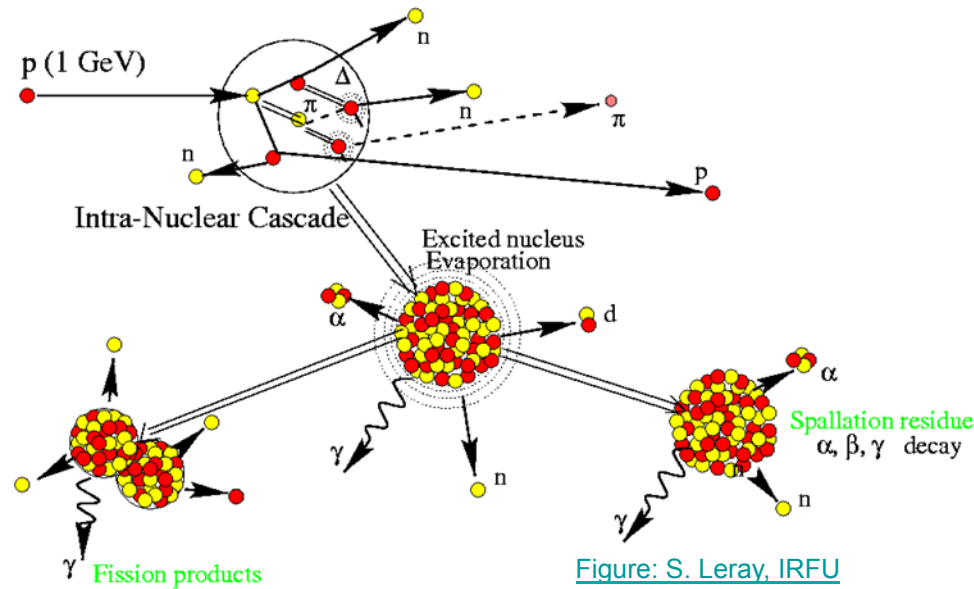


- Neutron transport code MCNP: Simulation of **neutron scattering** inside the neutron source and all surrounding materials e.g. collimators
- **Neutron scattering can change the correlation of time of flight and neutron energy**



- Unscattered neutrons can be identified (in the simulation)

Neutron production by spallation



Nucleon-Nucleus collisions at relativistic energies
(de Broglie wavelength < mean free path)
in two phases:

$T_{\text{coll}} < 10^{-22}\text{s}$:

Collisions of the projectile nucleon with nucleons in the target
(Intranuclear Cascade, emission of **fast** particles π, n, p, \dots)

$T_{\text{equil}} > 10^{-21}\text{s} - 10^{-16}\text{s}$

Reorganisation of the residual nuclei, thermalization,
particle evaporation (n, p, d, α, \dots), gamma ray emission

Spallation neutron yield

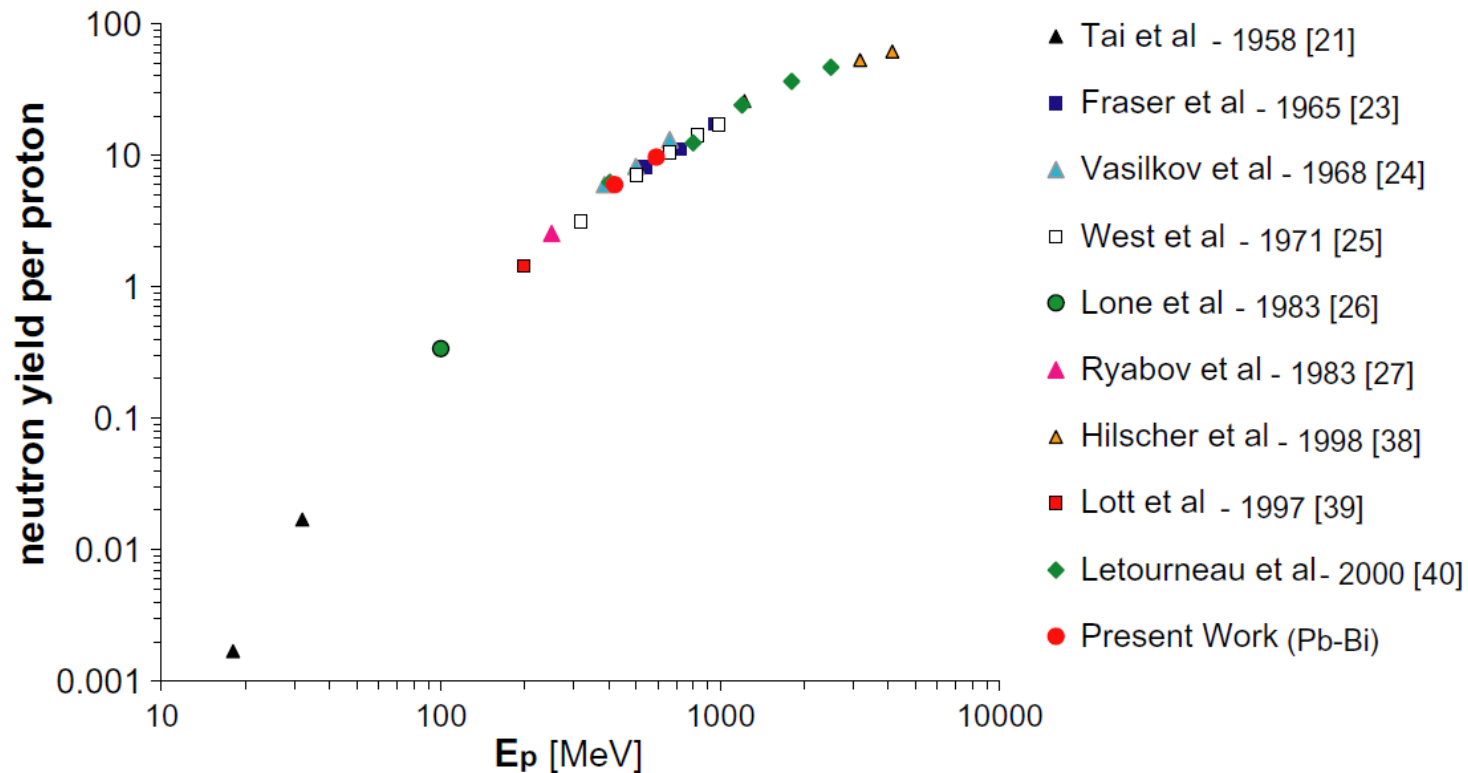
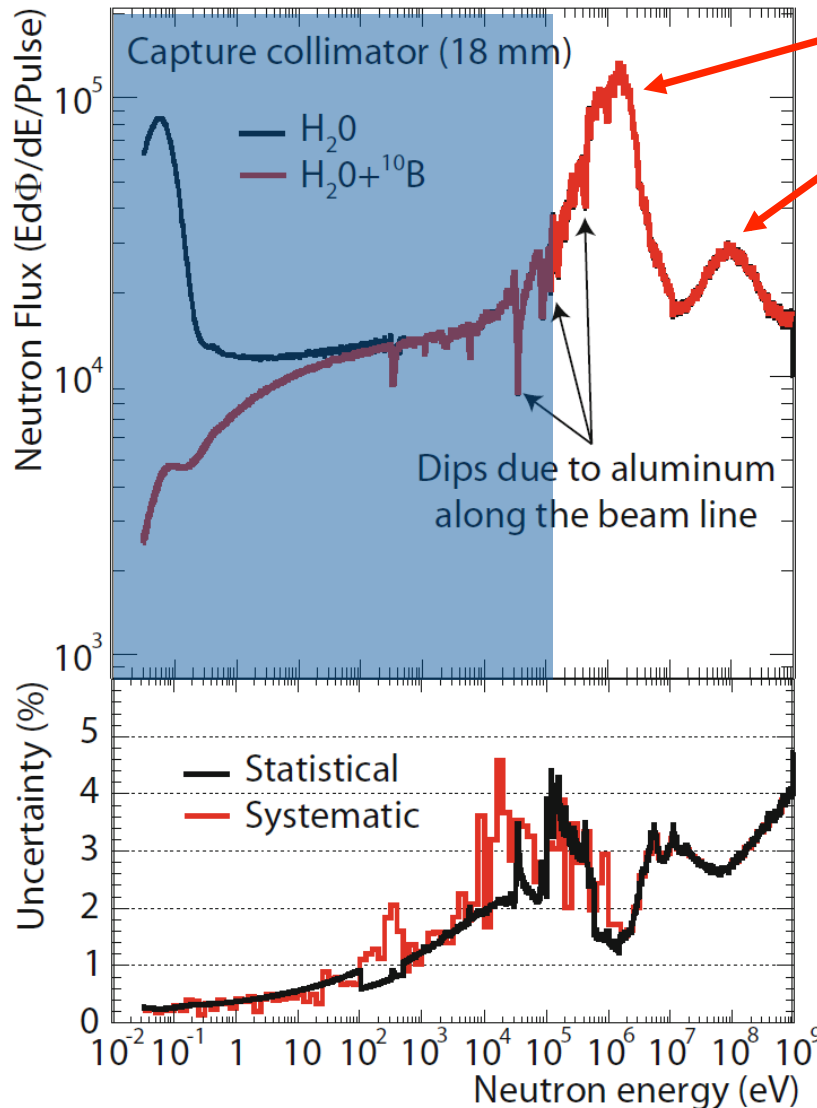


Fig. 10. Compilation of thick-target n/p values for p+Pb and Pb/Bi measured to date at all incident energies.

CERN nTOF ca. 300 n/p 20 GeV protons on Pb

Spallation neutron spectrum at CERN nTOF



Neutron evaporation (0.1 – 10 MeV)
Fast neutrons from intranuclear cascade stage > 10 MeV

Shaded range < 0.1 MeV
Neutrons slowed down by hydrogenous materials

CERN Proton Synchrotron 20 GeV/c

Slowing down of neutrons

- Neutron transport in general: (Boltzmann equation from statistical mechanics) describes the change of Neutron number density N because of a neutron flux density gradient, neutron sources, neutron absorption or in-scattering and out-scattering
- Fundamental physics used in neutron transport theory \rightarrow deterministic or stochastic (e.g. MCNP) methods to solve transport problems

$$\begin{aligned} \frac{\partial N(\vec{r}, \vec{\Omega}, E_n)}{\partial t} &= \frac{1}{\langle v_n \rangle} \frac{\partial \Phi(\vec{r}, \vec{\Omega}, E_n)}{\partial t} \\ &\quad - \vec{\Omega} \cdot \vec{\nabla} \Phi(\vec{r}, \vec{\Omega}, E_n) - \Sigma_t \Phi(\vec{r}, \vec{\Omega}, E_n) \\ &\quad + \int_{4\pi} \int_0^\infty \Sigma_s(\vec{\Omega}' \rightarrow \vec{\Omega}, E'_n \rightarrow E_n) \Phi(\vec{r}, \vec{\Omega}', E'_n) dV' d\Omega' dE'_n \\ &\quad + S(\vec{r}, \vec{\Omega}, E_n) \end{aligned}$$

- Q: How can we do simple tests to trust the simulation ?
A: Simple estimates using averaged parameters and compare results ...

Neutron transport by elastic scattering

- Neutron in a scattering medium
no absorption, no energy change by collisions
- $x \downarrow 0 = \lambda \downarrow s$ from the start the neutron moves on average the mean free path $\lambda \downarrow s$ until first collision
- $x \downarrow 1 = \lambda \downarrow s \cos \vartheta \downarrow 1 = \lambda \downarrow s \mu$ projection of the path traveled in the original direction

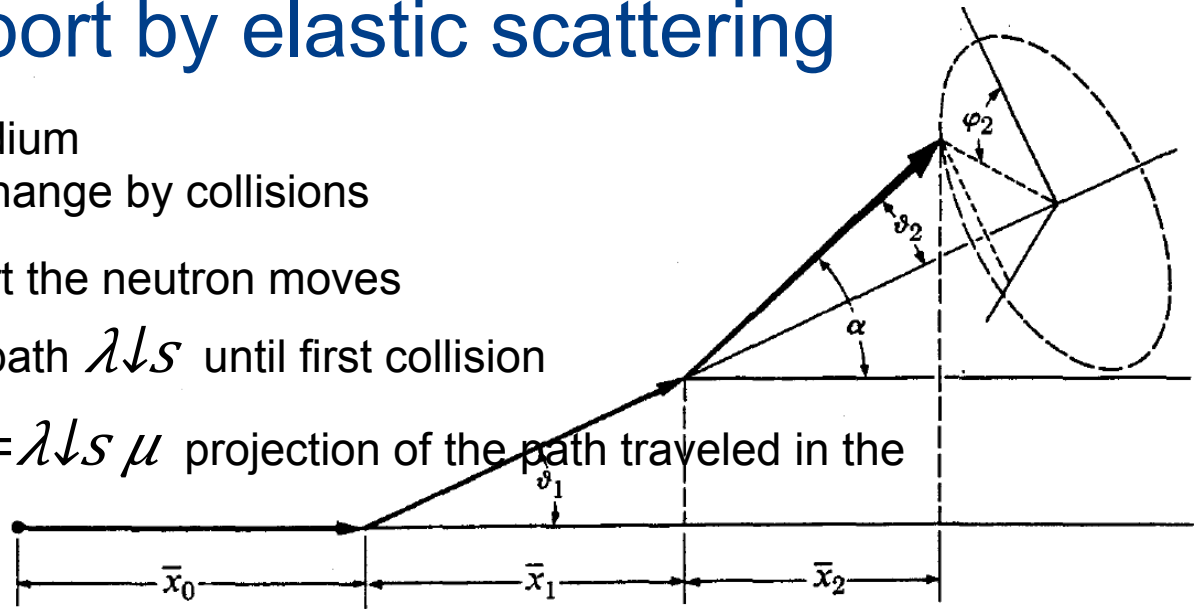


Fig. 2-22. Diagram for calculating the transport cross section.

- μ is the average value of the cosine of the scattering angle
 - At the second collision: $x \downarrow 2 = \lambda \downarrow s \cos \alpha$ and $\cos \alpha = \cos \vartheta \downarrow 1 \cos \vartheta \downarrow 2 + \sin \vartheta \downarrow 1 \sin \vartheta \downarrow 2 \cos \varphi \downarrow 2$
 - All values of $\varphi \downarrow 2$ are equally probable (average value of $\sin \vartheta \downarrow 1 \sin \vartheta \downarrow 2 \cos \varphi \downarrow 2 = 0$)
- The average of the mean free path projection in the direction of motion approaches zero. *The neutron forgets its original direction of motion.*

Neutron transport by elastic scattering

- The transport mean free path $\lambda_{tr} = \lambda_s / (1 - \mu)$ depends on μ the average value of the cosine of the scattering angle

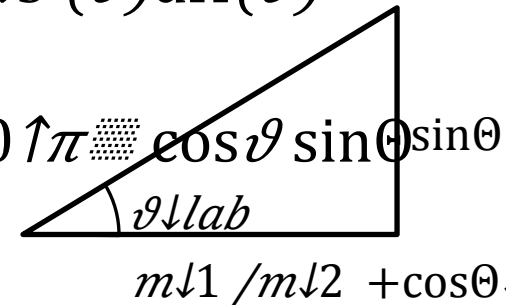
$$\mu = 1/\sigma_s \int 4\pi \sigma_s(\vartheta) \cos\vartheta d\Omega = 2\pi/\sigma_s \int_0^\pi \sigma_s(\vartheta) \cos\vartheta \sin\vartheta d\vartheta =$$

- For homogeneous media the scattering is **isotropic in the c.m. system**

up to $E_n \approx 10$ MeV. The differential cross section is a constant $\sigma_s(\Theta) = \sigma_s / 4\pi$

Transformation into the C.M. system $\sigma_s(\Theta) d\Omega(\Theta) = \sigma_s(\vartheta) d\Omega(\vartheta)$

$$\mu = 2\pi/\sigma_s \int_0^\pi \sigma_s(\Theta) \cos\vartheta \sin\Theta d\Theta = 1/2 \int_0^\pi \cos\vartheta \sin\Theta d\Theta$$



Relation of the scattering angle in the lab and c.m. system:

$$\cos\vartheta = 1 + A \cos\Theta / \sqrt{A^2 + 2A \cos\Theta + 1}$$

	H	D	C	Fe
μ	48°	70°	87°	89°

For heavy nuclei the scattering is isotropic only below 1 MeV

Simplified scattering kernel

- Probability distribution of scattered energies

- Assumption: Isotropic scattering in the c.m. system $\sigma_{\downarrow s}(\Theta) = \sigma_{\downarrow s} / 4\pi$

$$W(E \downarrow n \rightarrow E' \downarrow n') dE' \downarrow n' = -2\pi \sigma_{\downarrow s}(\Theta) \sin\Theta d\Theta / \sigma_{\downarrow s} \rightarrow -1/2 \sin\Theta d\Theta$$

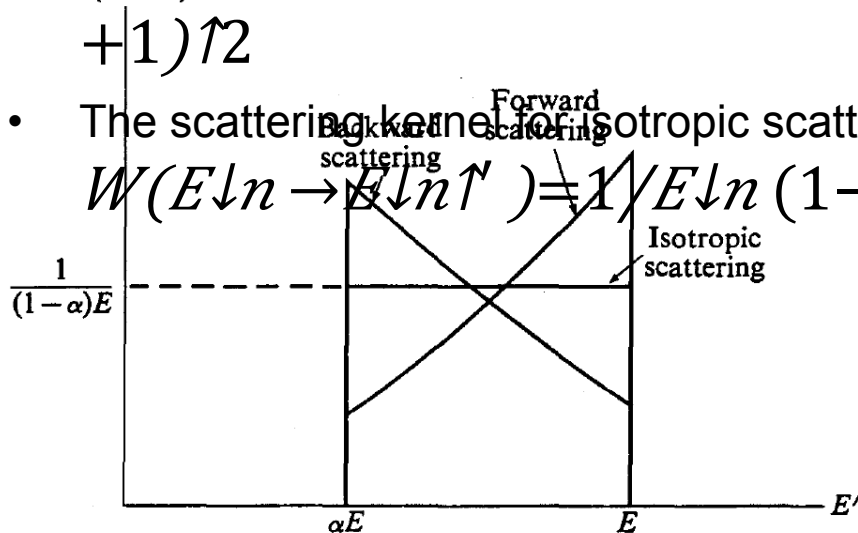
$$W(E \downarrow n \rightarrow E' \downarrow n') dE' \downarrow n' = -W(\Theta) d\Theta = -W(\Theta) d\Theta / dE' \downarrow n' dE' \downarrow n'$$

- The energy of the scattered neutron is a function of the scattering angle:

$$dE' \downarrow n' / d\Theta = 1/2 E \downarrow n (1-\alpha)(-\sin\Theta); \alpha \stackrel{\text{def}}{=} (A-1)^2 / (A+1)^2$$

- The scattering kernel for isotropic scattering is

$$W(E \downarrow n \rightarrow E' \downarrow n') = 1/E \downarrow n (1-\alpha) dE' \downarrow n'$$



Average energy loss in elastic scattering and average energy of elastically scattered neutrons

- With the scattering kernel the average energy loss is:

$$\langle \Delta E \rangle = \int_{E \downarrow \min}^{\uparrow E \downarrow \max} W(E \downarrow n \rightarrow E \downarrow n \uparrow') (E \downarrow n - E \downarrow n \uparrow') dE \downarrow n \uparrow'$$

$$\langle \Delta E \rangle = \int \alpha E \downarrow n \uparrow E \downarrow n \uparrow' \cdot \frac{1}{E \downarrow n} (1 - \alpha) (E \downarrow n - E \downarrow n \uparrow') dE \downarrow n \uparrow' = 1 - \alpha/2 E \downarrow n$$

- The relative energy loss is constant: $\langle \Delta E \rangle / E \downarrow n = 1 - \alpha/2$
- Average energy of elastically scattered neutrons:

$$\langle E \downarrow n \uparrow' \rangle = \int_{E \downarrow \min}^{\uparrow E \downarrow \max} W(E \downarrow n \rightarrow E \downarrow n \uparrow') (E \downarrow n \uparrow') dE \downarrow n \uparrow'$$

$$\langle E \downarrow n \uparrow' \rangle = E \downarrow n (1 + \alpha)/2$$

- The average energy of the scattered neutron is in the center of the range $[E \downarrow n, \alpha E \downarrow n]$

Further reading:

K. Wirtz, K. Beckurtz, Elementare Neutronenphysik, Springer 1958

And other literature about neutron transport theory

How many collisions are required to moderate a neutron ?

- Estimate using the lethargy $u \stackrel{\text{def}}{=} \ln E \downarrow n / E \downarrow n \uparrow$
- The average logarithmic energy loss is: $\xi = \int \alpha E \downarrow n \uparrow E \downarrow n \cdot W(E \downarrow n \rightarrow E \downarrow n \uparrow) \ln E \downarrow n / E \downarrow n$

Special case: isotropic scattering: $\xi(\alpha) = 1 + \alpha/1 - \alpha \ln \alpha$

- Average logarithmic energy loss ξ for hydrogen ($\alpha=0$ as $A=1$)

$$\lim_{\alpha \rightarrow 0} \xi(\alpha) = 1$$

- Assume initial lethargy zero. After n collisions increase of lethargy is

$$u \downarrow n = \sum_{i=1}^n \Delta u \downarrow i$$

- The average lethargy after n collisions is similar

$$u \downarrow n = \sum_{i=1}^n \Delta u \downarrow i$$

- The average lethargy $\Delta u \downarrow i = \xi \rightarrow n = u \downarrow n / \xi$

- To increase the average lethargy to the value $u \downarrow n$ whose corresponding energy is E

$$n = 1/\xi \ln E \downarrow 0 / E$$

collisions are required.

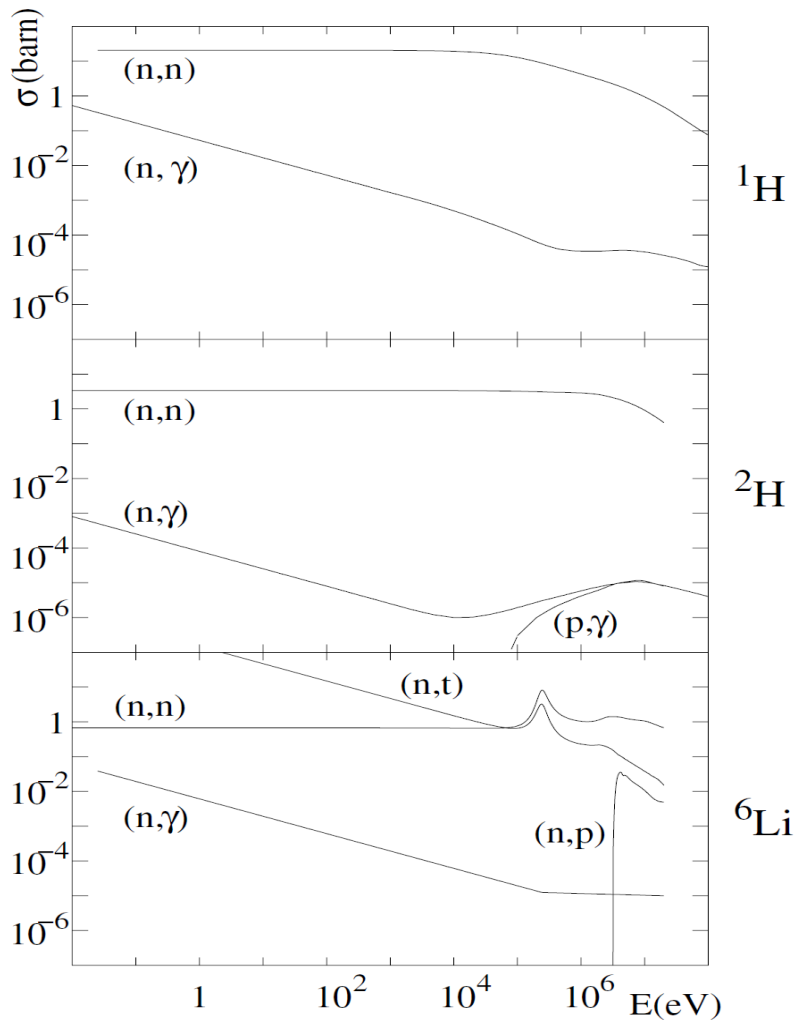
Average number of collisions to slow down neutrons ...

Nuclide	A	α	ξ	n
H	1	0	1	18
D	2	0.111	0.725	25
Be	9	0.64	0.209	86
C	12	0.716	0.158	114
Fe	56	0.931	0.0353	516
U	238	0.983	0.00838	2172

- Estimated with $n = 1 / \xi \ln E \downarrow 0 / E$ assuming isotropic scattering
 $\xi(\alpha) = 1 + \alpha / (1 - \alpha \ln \alpha)$
 from $E_n = 2 \text{ MeV}$ to 25 meV

Slowing down vs. Neutron capture

116 3. Nuclear reactions



Elastic scattering on light nuclei
Efficiently slows neutrons down.

In normal hydrogen neutron capture
reduces the neutron flux density
($E_\gamma = 2.2$ MeV capture gamma rays)

In deuterium the capture fraction is
Much lower

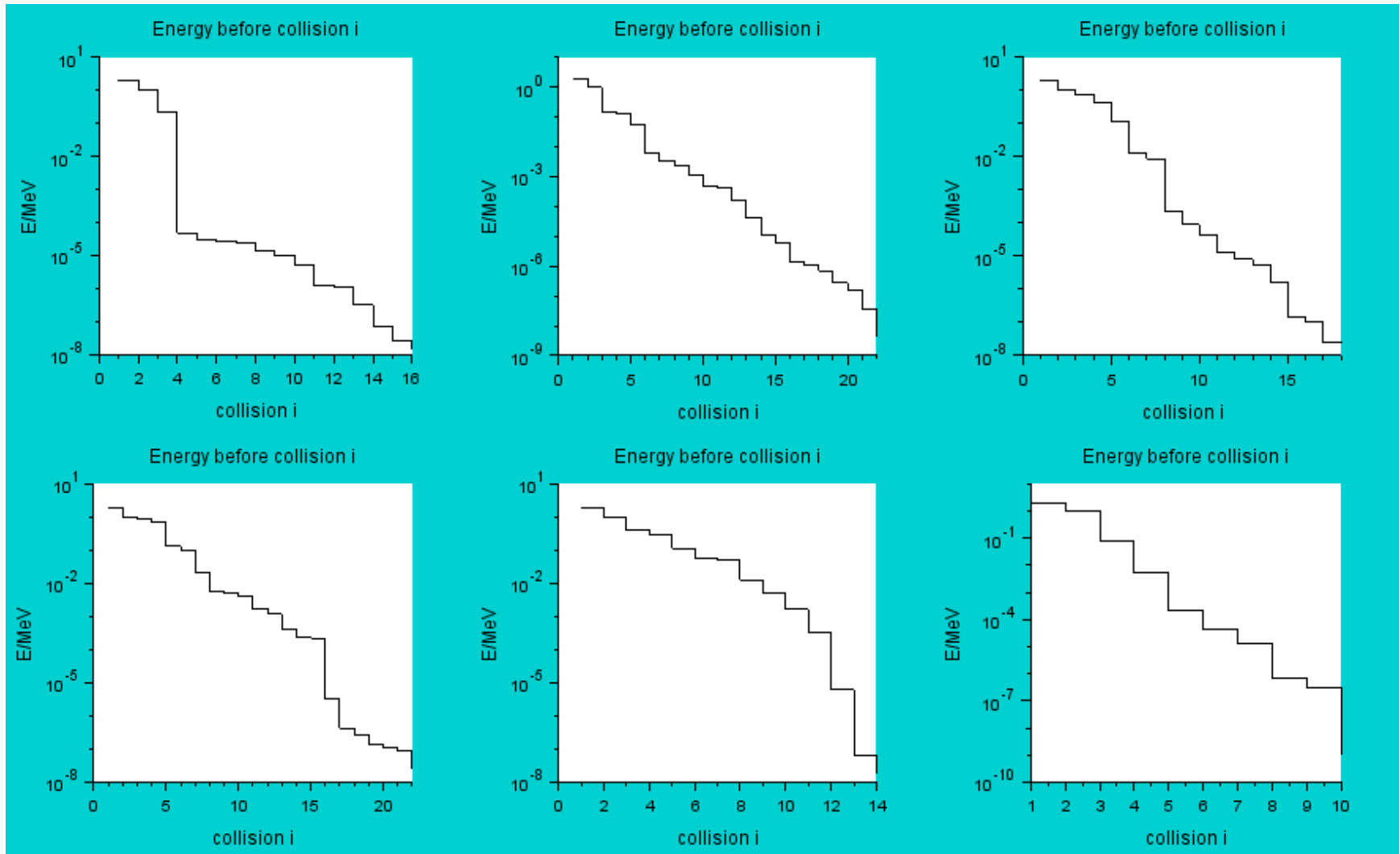
Fig. 3.4. Examples of reaction cross-sections on ^1H , ^2H , and ^6Li [30]. Neutron elastic scattering, (n,n), has a relatively gentle energy dependence while the exothermic reactions, (n, γ) and $^6\text{Li}(n,t)^4\text{He}$ ($t=\text{tritium}=\text{}^3\text{H}$), have a $1/v$ dependence at low energy. The exothermic (p, γ) reaction is suppressed at low energy because of the Coulomb barrier. The reaction $^6\text{Li}(n,p)^6\text{Be}$ has an energy threshold. The fourth excited state of ^7Li (Fig. 3.5) appears as a prominent resonance in n ^6Li elastic scattering and in $^6\text{Li}(n,t)^4\text{He}$.

Simulation of neutron random-walk

- Monte Carlo Simulation of neutrons slowing down in hydrogenous material e.g. Polyethylene (only hydrogen atoms take part) programmed in [scilab](#) by Georg Schramm, HZDR (similar to Matlab, Octave with graphics capabilities)
1. Neutrons start all in the same direction or isotropically.
 2. Isotropic scattering angles φ, ϑ sampled for each collision
 3. Calculation of energy and lab angle from kinematics relations
 4. Mean free path random samples up to $5\lambda\downarrow s$ from an exponential-distribution with average value $\lambda\downarrow s$
 5. Energy dependent n-p scattering cross section formula.
 6. Statistical analysis of the MC results

Neutron energy loss from elastic scattering

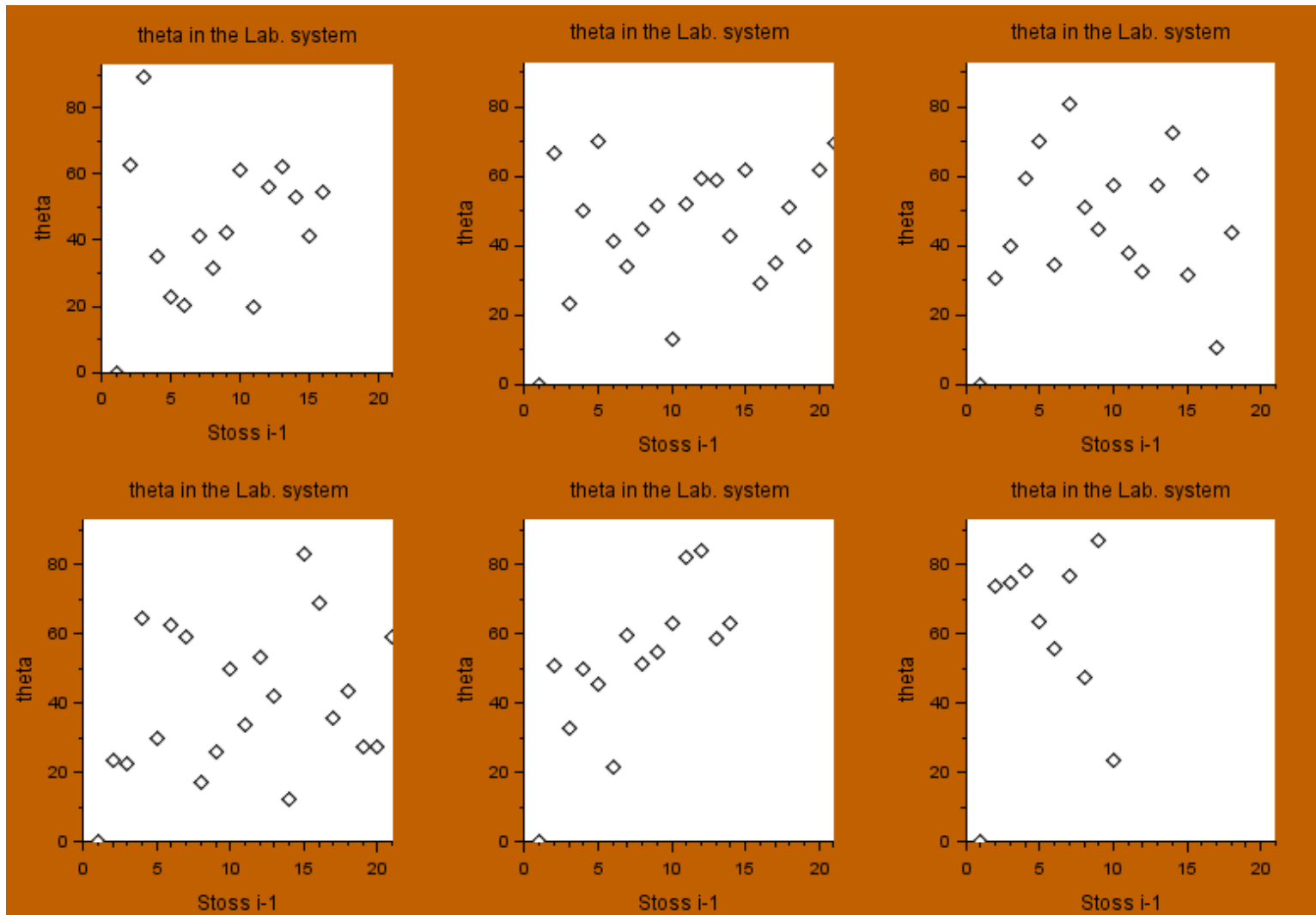
Six simulated neutron trajectories



$E_n = 2 \text{ MeV}$ down to $E_n = 25 \text{ meV}$

Scattering angle in the lab from elastic scattering

Six simulated neutron trajectories



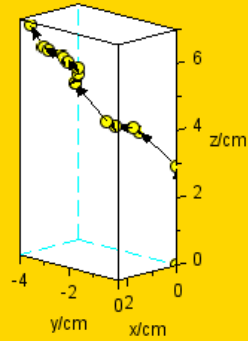
$E_n = 2 \text{ MeV}$ down to $E_n = 25 \text{ meV}$

Scattering angle in the lab from elastic scattering

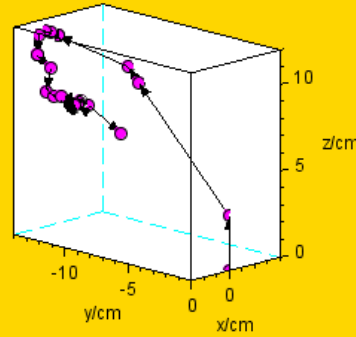
Six simulated neutron trajectories.

All neutrons start in the same direction along the positive z-axis.

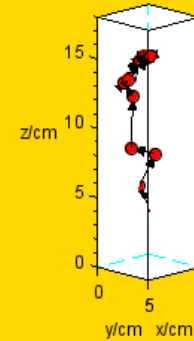
Neutron trajectory



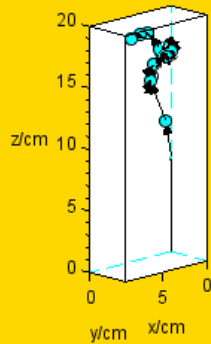
Neutron trajectory



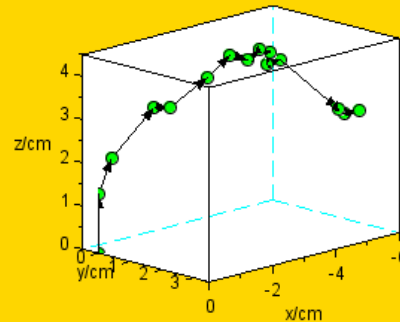
Neutron trajectory



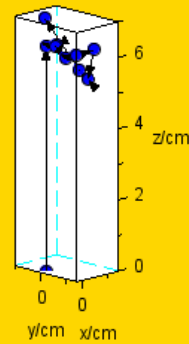
Neutron trajectory



Neutron trajectory



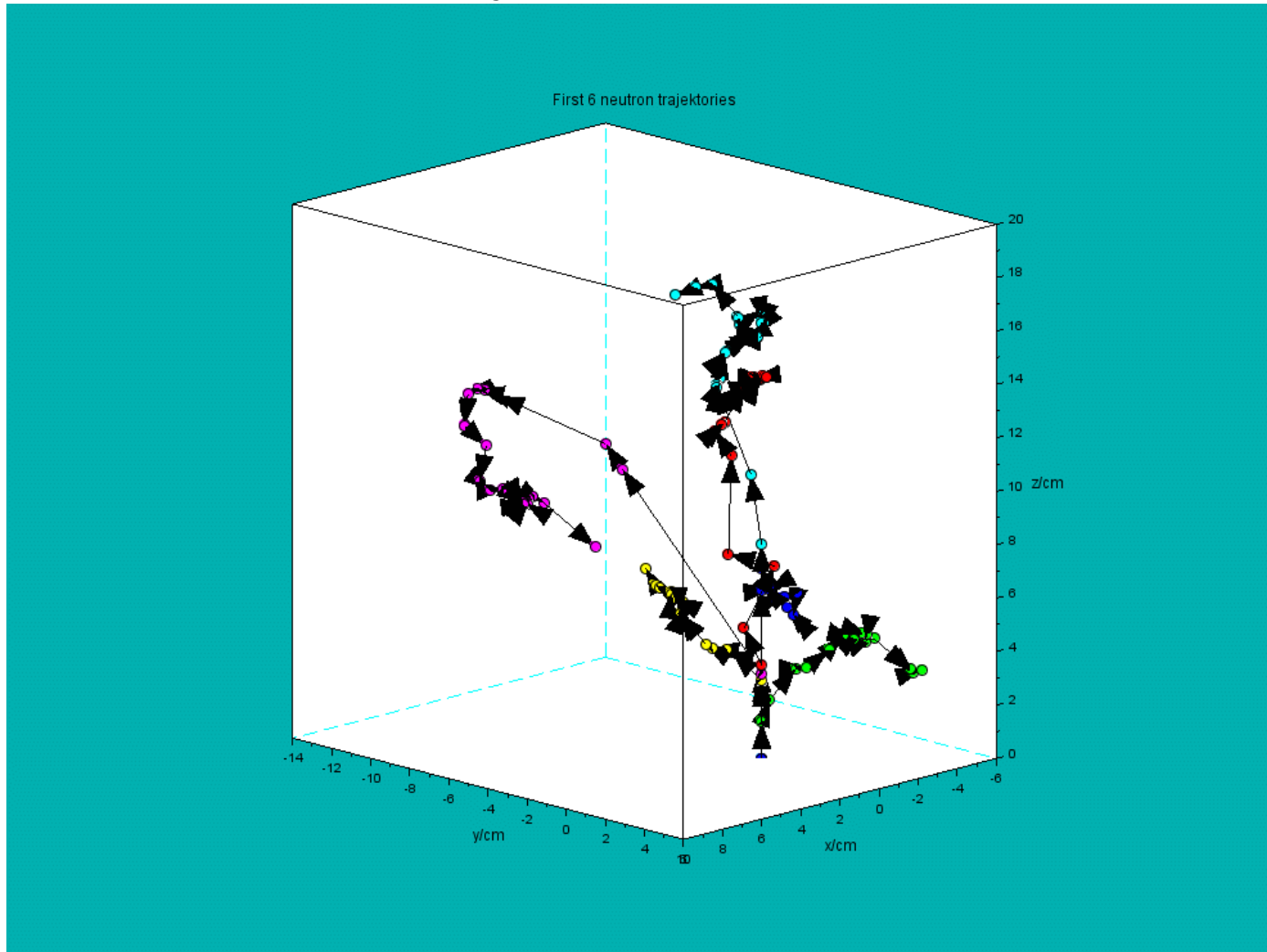
Neutron trajectory



$E_n = 2 \text{ MeV}$ down to $E_n = 25 \text{ meV}$

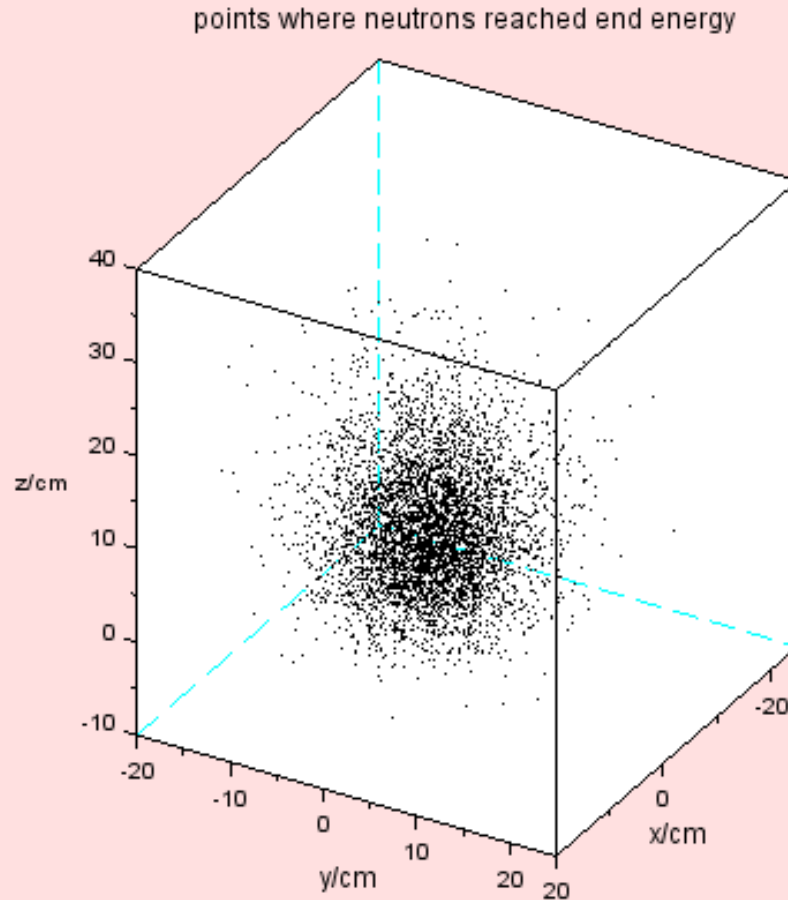
Scattering angle in the lab from elastic scattering

Six simulated neutron trajectories



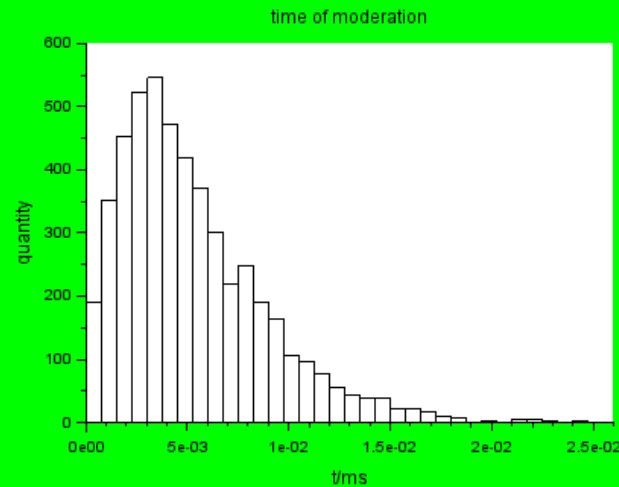
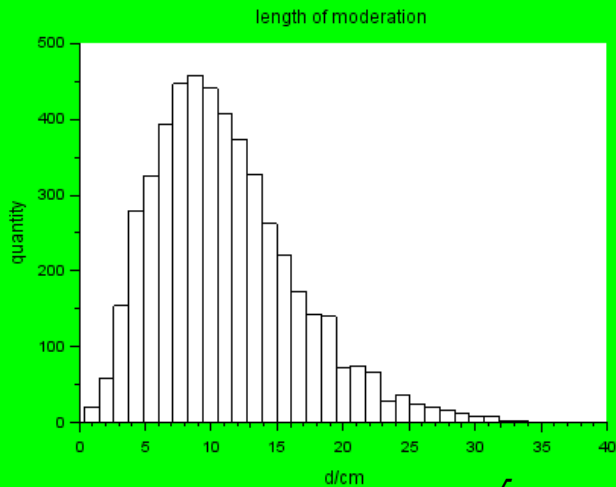
$E_n = 2 \text{ MeV}$ down to $E_n = 25 \text{ meV}$

Points where neutrons reach thermalisation

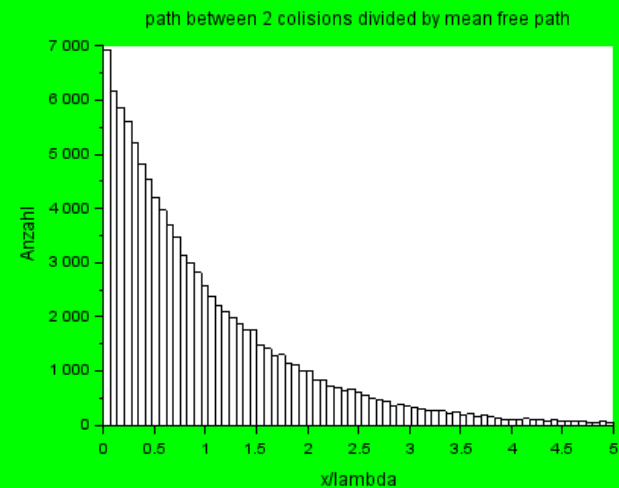
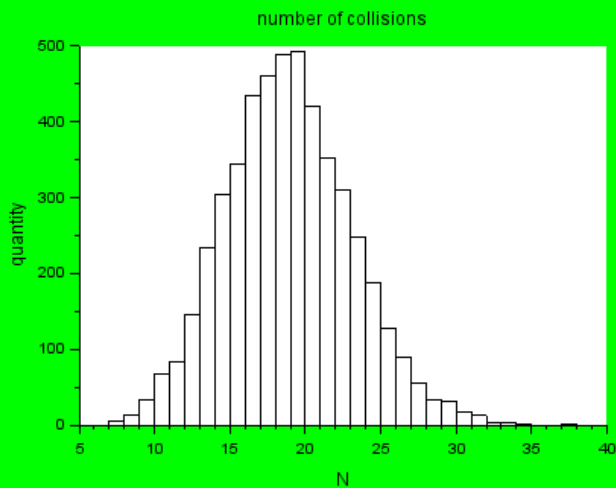


$E_n = 2 \text{ MeV}$ down to $E_n = 25 \text{ meV}$
Neutrons have lost their initial direction.

Slowing down distance, time of moderation, number of collisions, mean free



RMS slowing down distance $\sqrt{R12} = 11.1$ cm Average slowing down time: $5.3 \mu\text{s}$



Average number of collisions: 19.4

Estimates of the slowing down distance / time

$$\langle R^2 \rangle = 2 \cdot \left[\sum_{j=1}^n \lambda_j^2 + \sum_{j=1}^{n-1} \lambda_j \sum_{k=j+1}^n \lambda_k \langle \cos \theta \rangle^{k-j} \right]$$

$$\bar{t} = \sum_{j=1}^n \Delta t_j = \sum_{j=1}^n \frac{\lambda_j}{v_j}$$

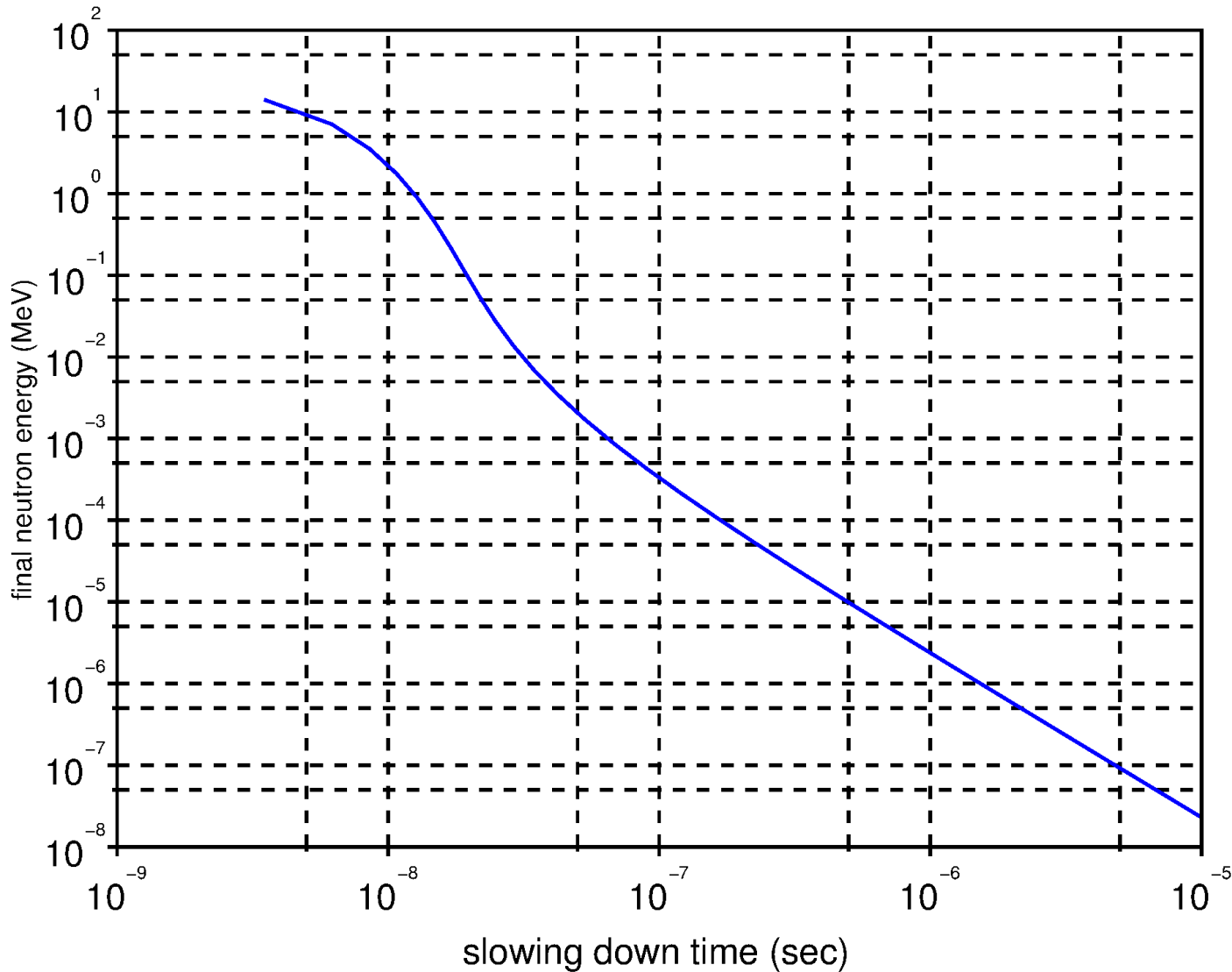
E. Fermi assumed constant mean free path: (not realistic for fast neutrons)

$$\langle R^2 \rangle = 2\lambda^2 \cdot \left[\sum_{j=1}^n 1 + \sum_{j=1}^{n-1} \sum_{k=j+1}^n \langle \cos \theta \rangle^{k-j} \right]$$

$$\langle R^2 \rangle = \frac{2n\lambda^2}{1-\mu} \left[1 - \frac{\mu}{n} \frac{1-\mu^n}{1-\mu} \right]$$

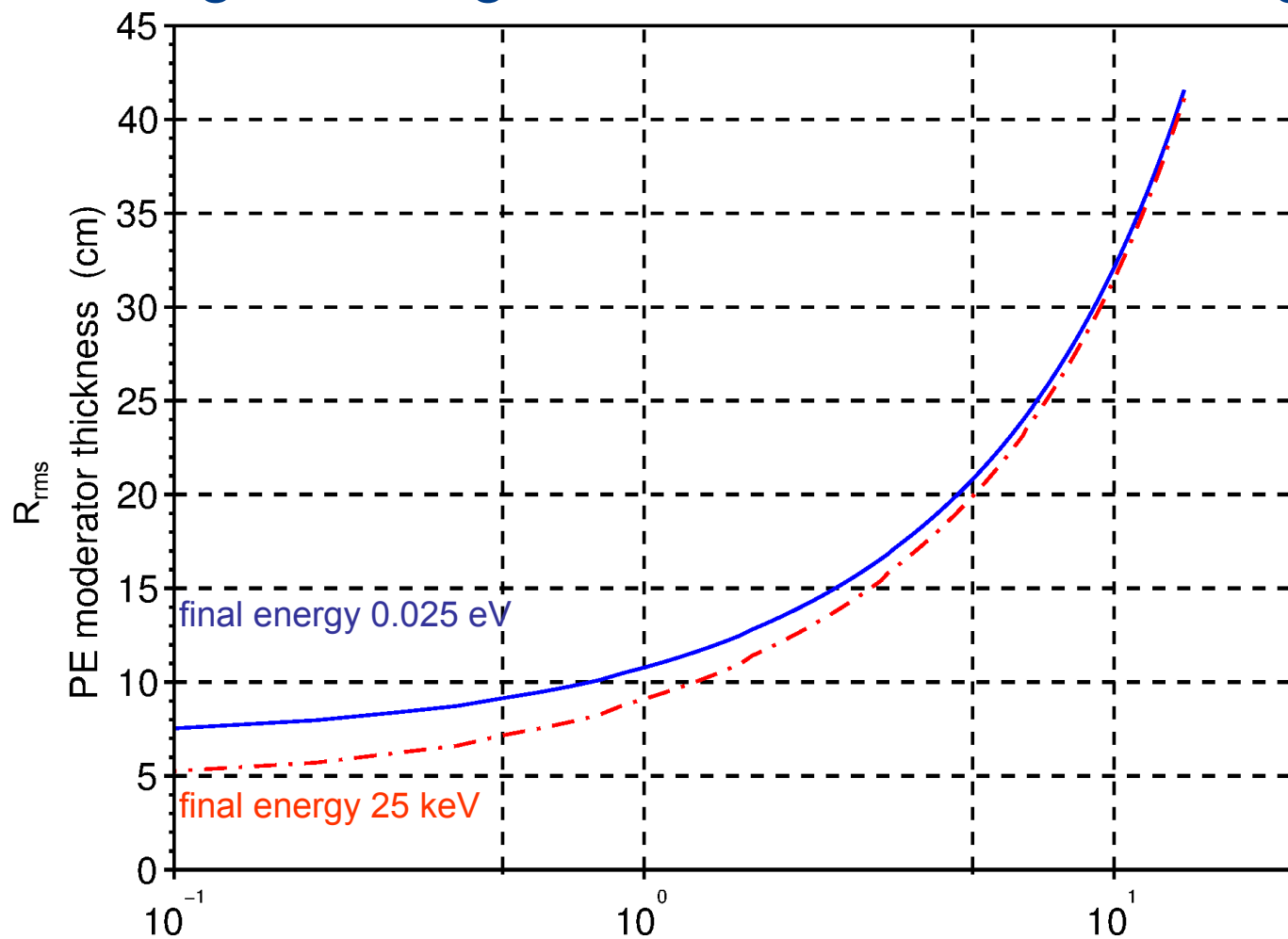
For a comparison of „averaged“ analytical description of the random-walk and full MC
See W.J. Nellis, Am.Jour.Phys. 45 (1977) 444

Average slowing down time of 14.1 MeV Neutronen:



To reach thermal energy from 14 MeV takes several μs

Average slowing down distance from averaged parameters



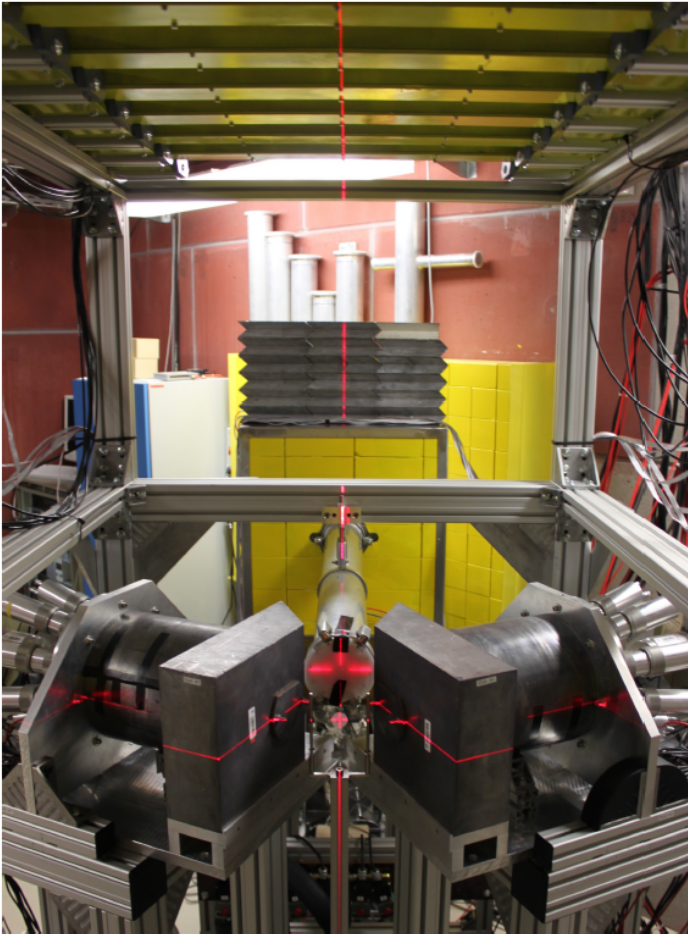
Average slowing down distance depends only weakly on the final energy required.

11 cm PE thermalize a 1 MeV Neutron
41 cm PE thermalize a 14.1 MeV Neutron

Shielding and collimation of neutrons

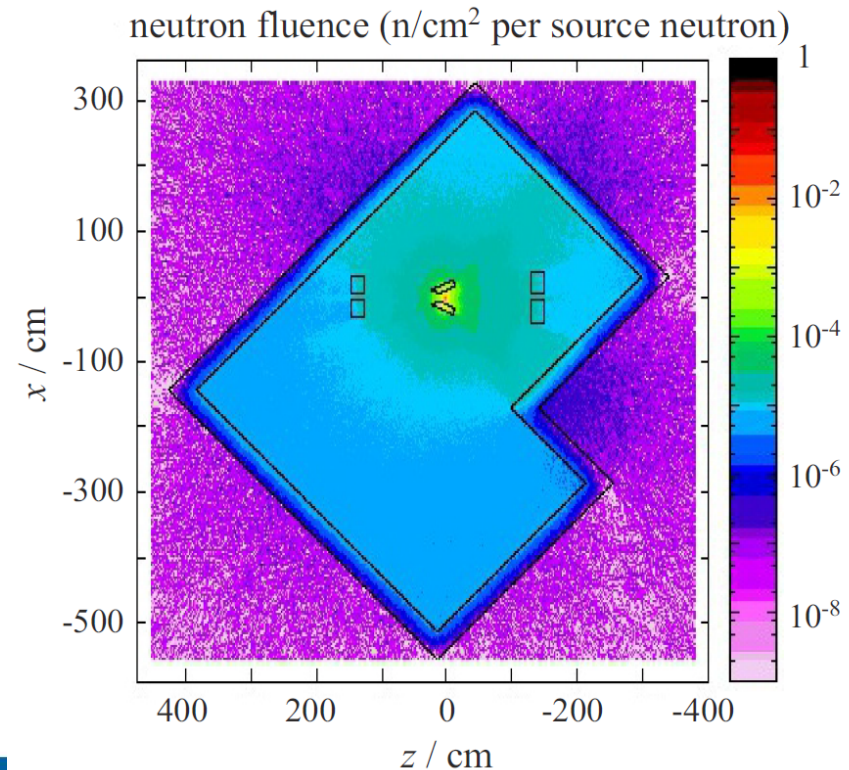
- Shielding and collimations of neutrons in the fast energy range **is complicated** and can even be **very** complicated.
- The complete experimental setup in a rather realistic geometry needs to be simulated with reliable particle transport simulations.
- A shielding needs to be thicker than the slowing down distance discussed here in order to reach an intensity reduction of 10^{-3} to 10^{-6} depending on the quality required.
- In general, all material very close to the neutron source or detector causes the most scattering and experimental background due to the large geometrical solid angle.
- Room return background of scattered neutrons from the walls, floor and ceiling is difficult to avoid and usually not negligible.
- Monte Carlo simulations for shieldings and collimators are not straightforward as very high statistical accuracy is needed to describe strong intensity reductions of several orders of magnitude.
- Simple attenuation estimates can help to test if any biasing methods in the MC simulation are adequate

Neutron scattering in a deuteron breakup exp.



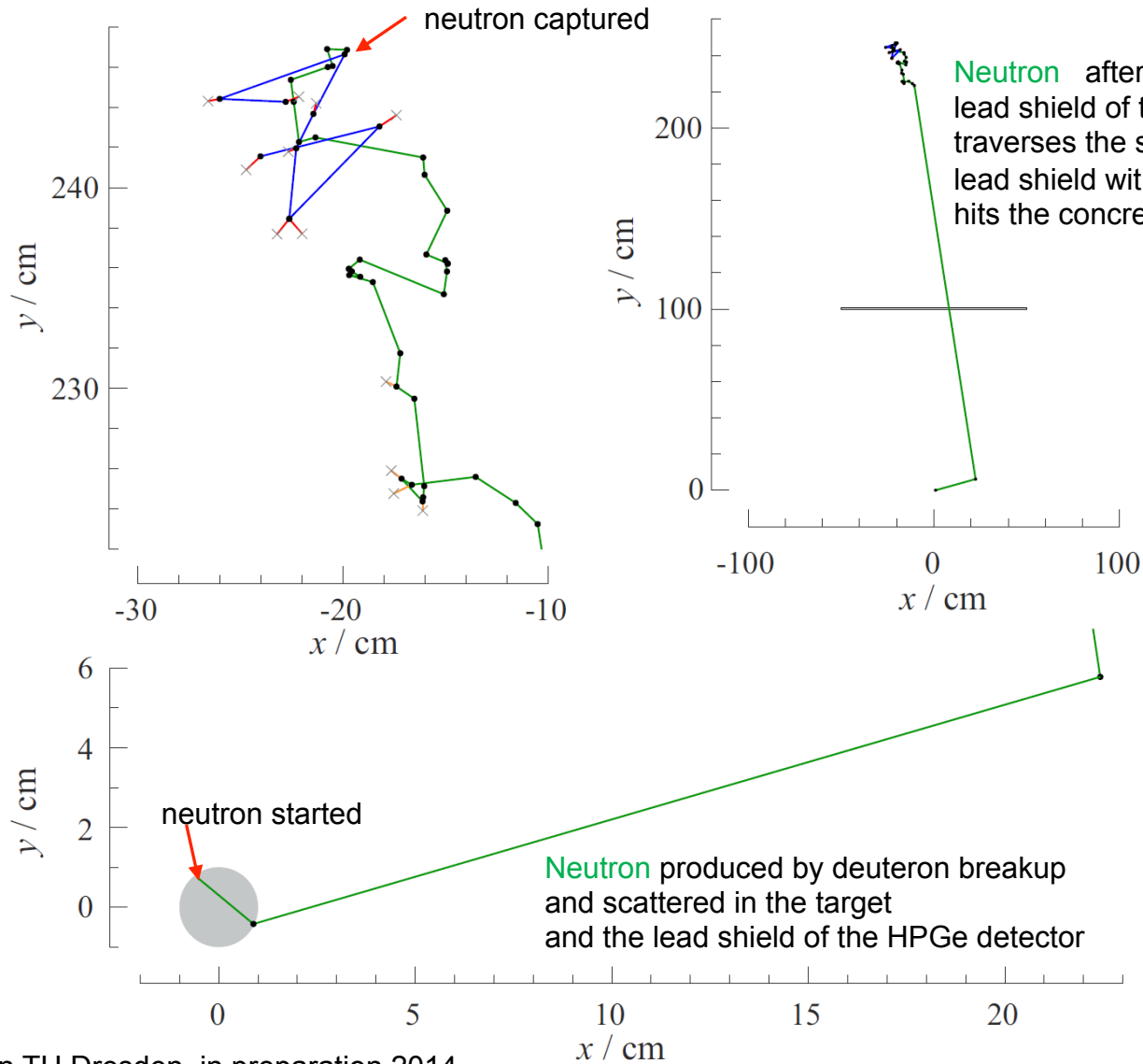
$D(\gamma, n)H$ cross section measurement:
Time of flight measurement of breakup neutrons with plastic scintillators
Bremsstrahlung intensity determined by nuclear resonance fluorescence of ^{27}Al with HPGe detectors
Target 2 cm thick CD_2 with Al disks
pulsed bremsstrahlung (615 ns); end point energy 6 MeV

FLUKA simulation of the complete setup



Typical neutron trajectory

3 Projections in the vertical x-y plane of the same simulated event



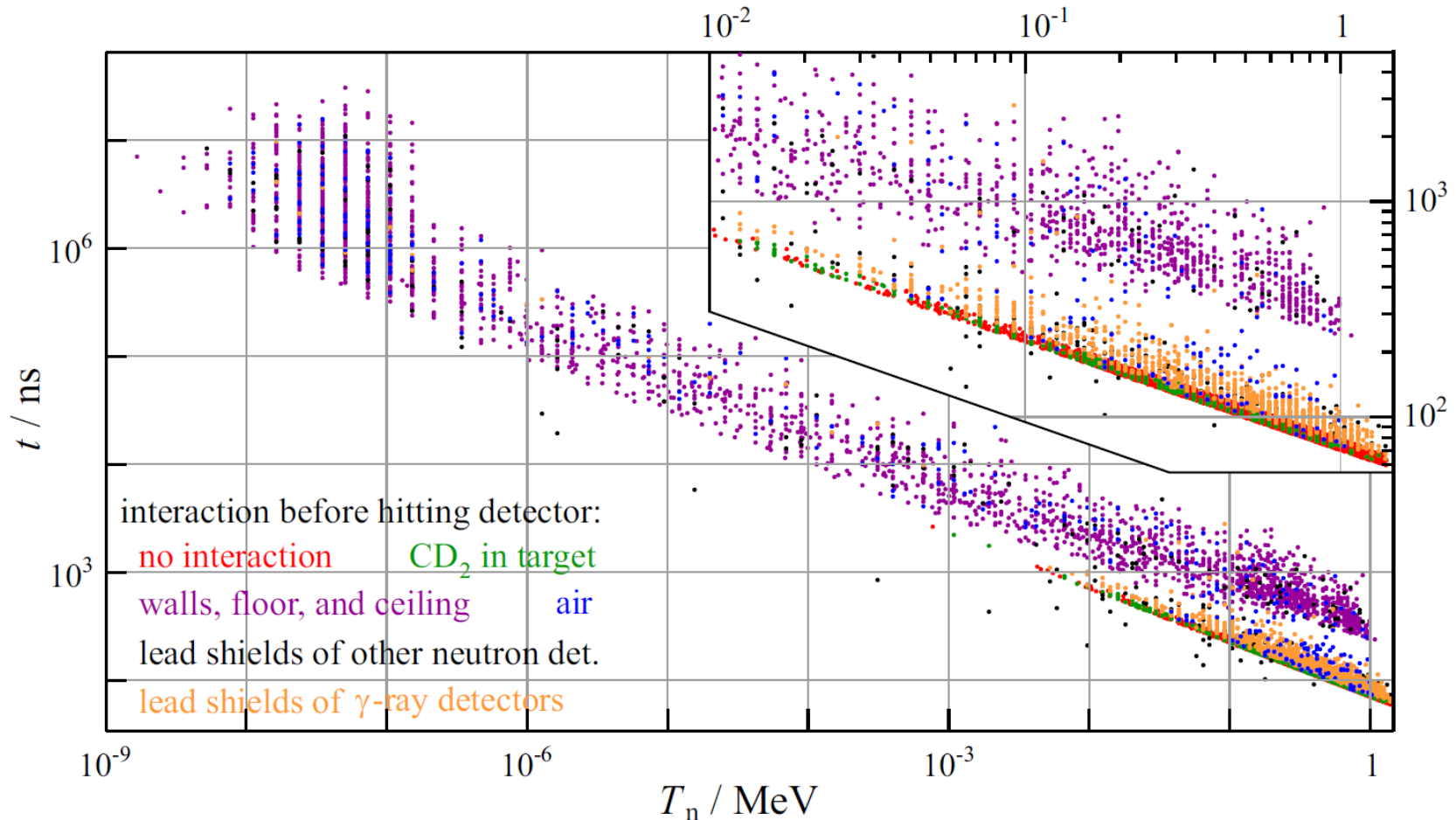
Neutron is slowed down in the ceiling by elastic scattering (hydrogen)
Two capture gamma-rays scattering on atoms in the wall

Neutron after scattering in the lead shield of the HPGe, traverses the scintillator and it's lead shield without interaction and hits the concrete ceiling.

neutron started

Neutron produced by deuteron breakup and scattered in the target and the lead shield of the HPGe detector

Time of flight to neutron energy correlation

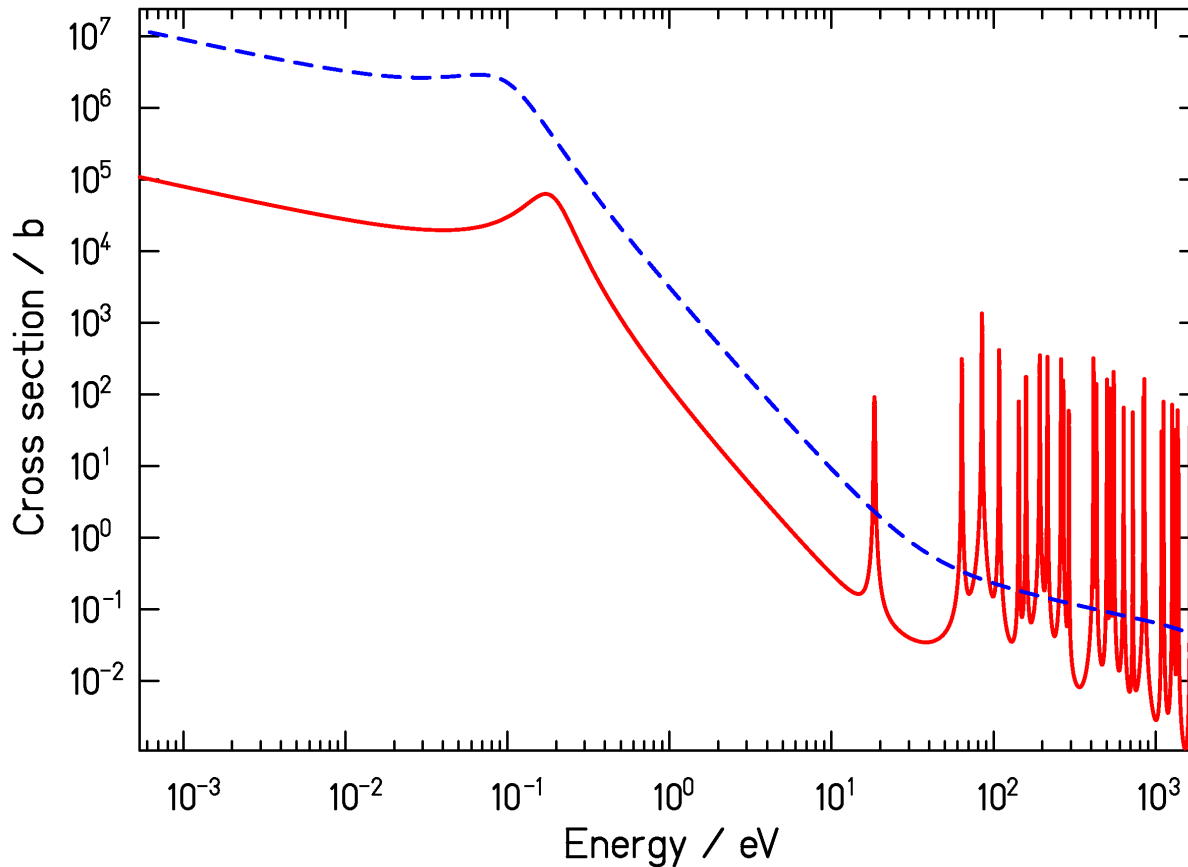


1. scattering of neutrons in the MeV range \rightarrow room return
2. Lead shield around HPGe is detrimental to the tof measurement.

The inset shows the range where the used detectors had sufficient efficiency

Strong neutron absorption through resonances

Joint Evaluated Fission and Fusion Data



Not many resonances known in ^{135}Xe

— $^{113}\text{Cd}(n,\gamma)$
 - - - $^{135}\text{Xe}(n,\gamma)$

level spacing in ^{113}Cd
 ca. 24.8 eV
 probability for a resonance
 between 0-0.2 eV :
 $0.2 \text{ eV} / 24.8 \text{ eV} = 0.8 \% (!)$

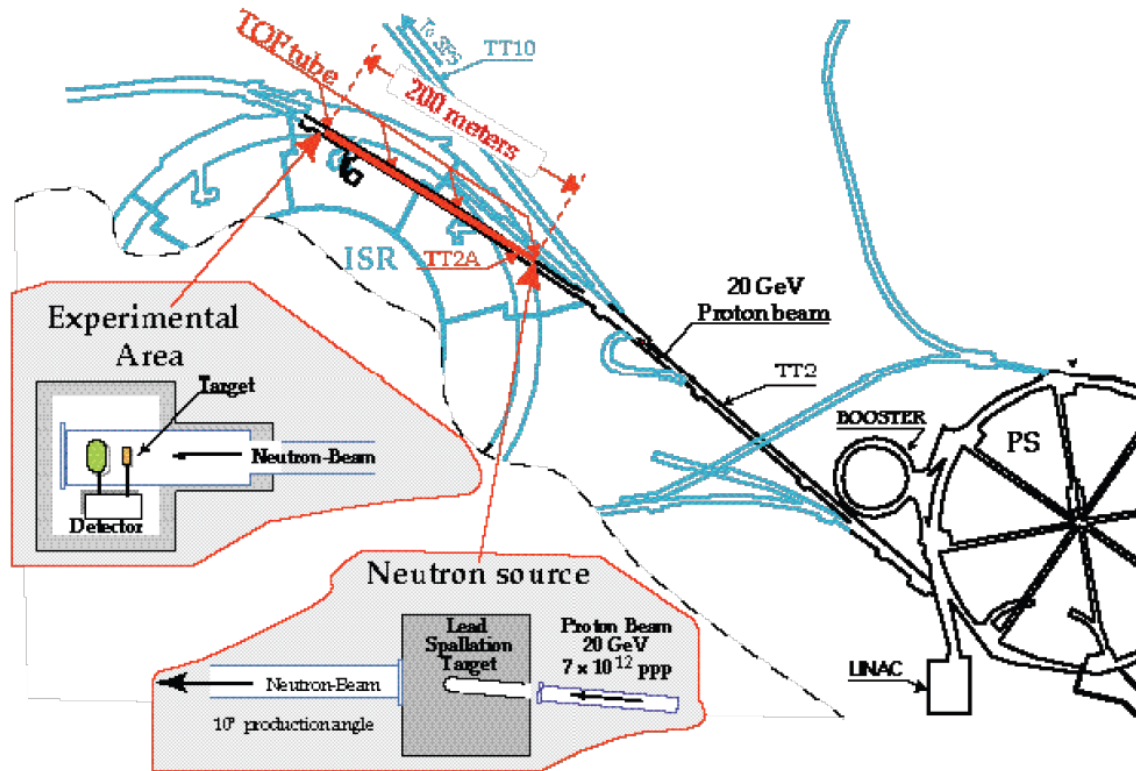
Neutrons are slowed down by elastic scattering and then captured

The radiative neutron capture cross section rises as $1/v \downarrow n$

In some nuclides strong resonances are located close to the thermal range

e.g. $^{113}\text{Cd}+n$ $\sigma \downarrow n, \gamma = 20600$ barn, ^{135}Xe , ...

CERN n_TOF Experiment



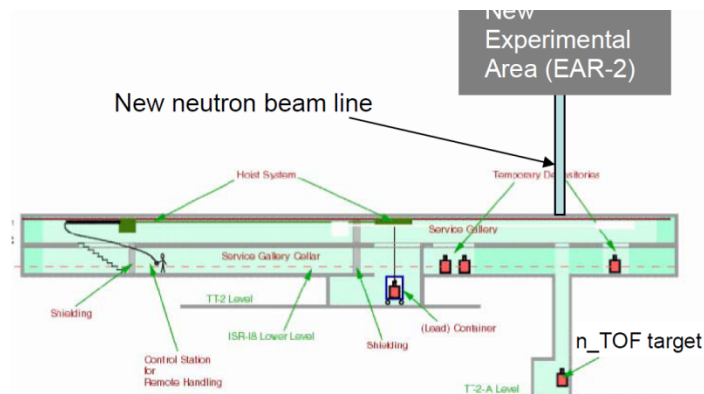
- New spallation target in 2009
- ca. 300 n per proton of 20 GeV
- Radioactive target capability at experimental station

Scientific programme:
nuclear astrophysics
(neutron capture)

Nuclear Data measurements
(neutron induced fission)

[CERN nTOF performance report](#)
CERN/INTC-O-011
INTC-2002-037
CERN-SL-2002-053 ECT

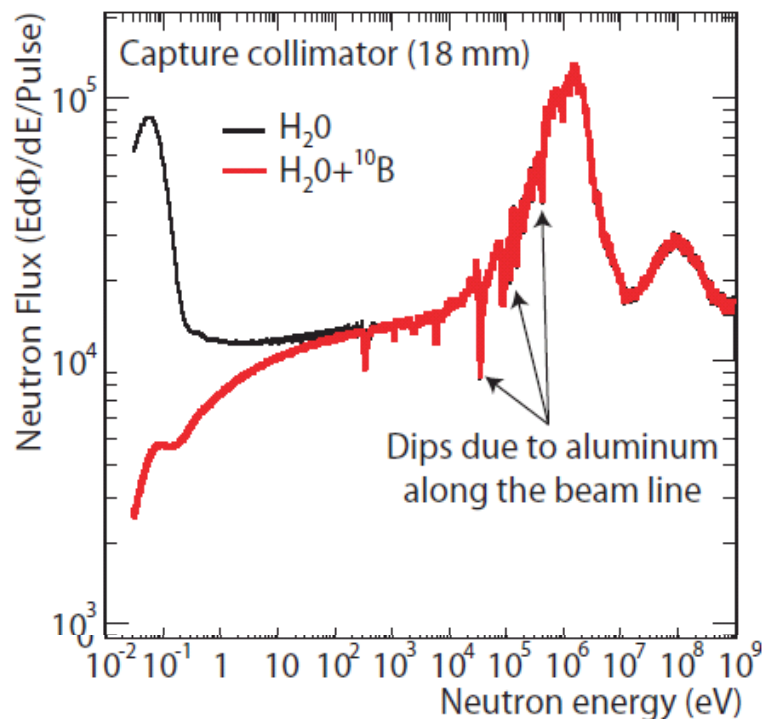
n_TOF Experimental Area 2 (EAR-2)
[CERN-INTC-2012-029](#) / INTC-O-015
will be operational in July 2014



EAR-2:
short flight path 20 m
for higher intensity
90° to the proton beam
→ Background reduction

Main characteristics of the existing n_TOF EAR-1

NEUTRON FLUX

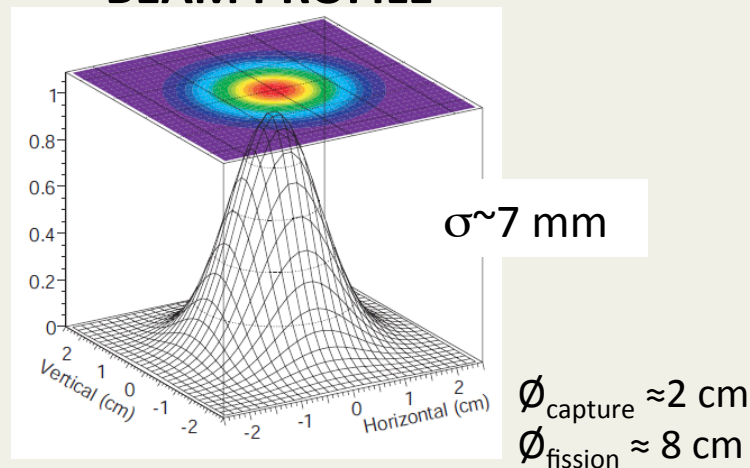


$0,6 \cdot 10^6$ neutrons/pulse (capture mode)

$12 \cdot 10^6$ neutrons/pulse (fission mode)

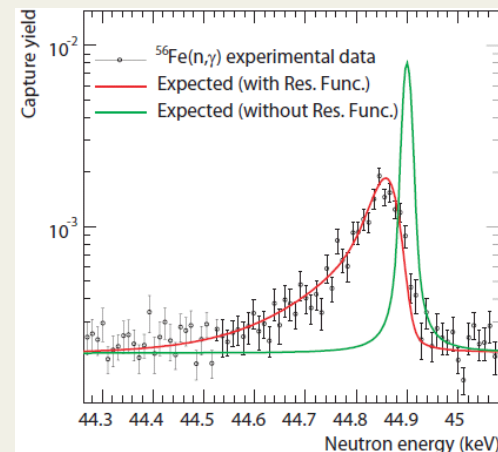
See [C. Guerrero et al. Eur Phys. J. A 49:27 \(2013\)](#)

BEAM PROFILE

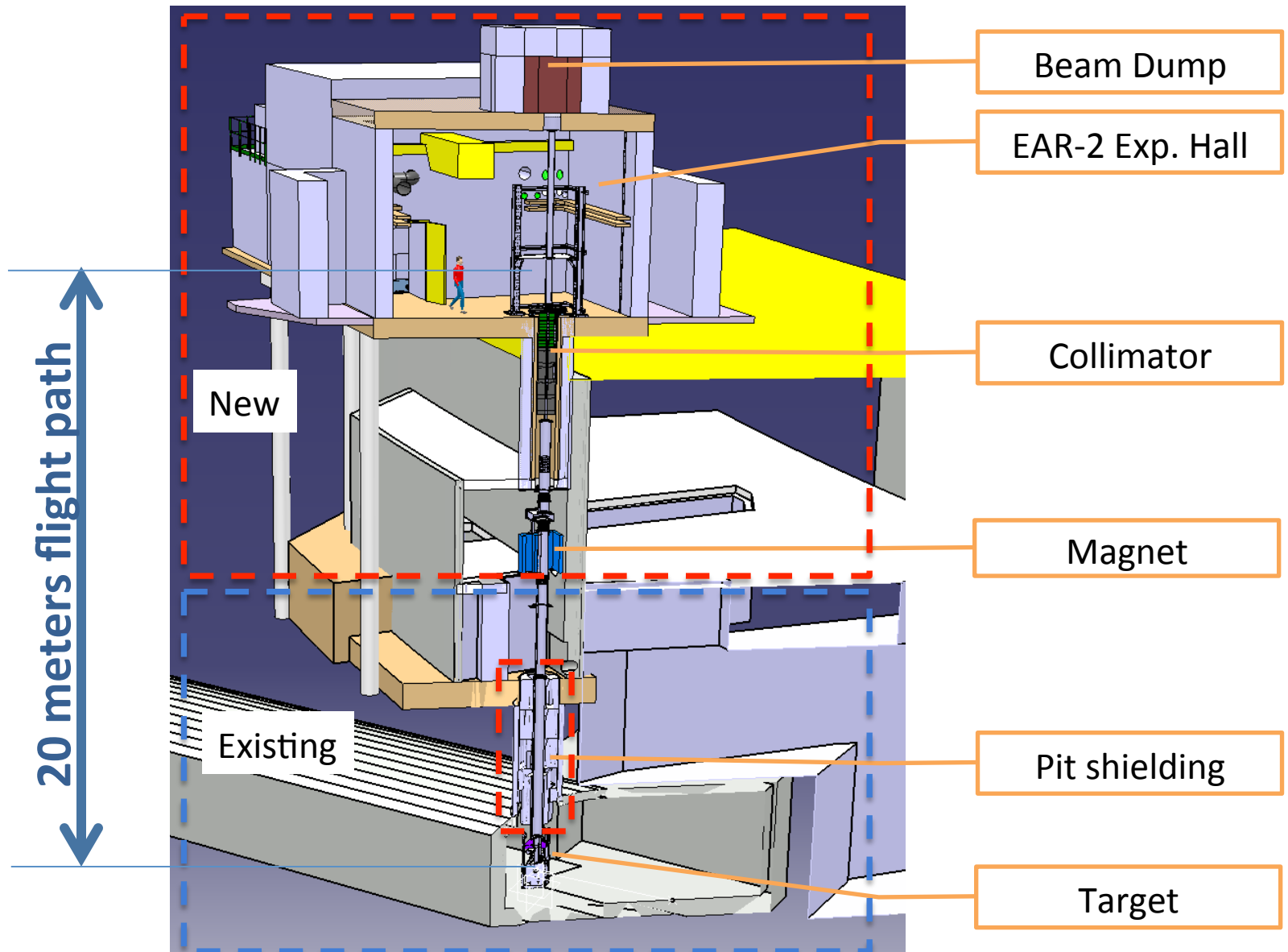


ENERGY RESOLUTION

E_n (eV)	$\Delta E_n/E_n$
1	$4.3 \cdot 10^{-4}$
10	$4.3 \cdot 10^{-4}$
10^2	$4.3 \cdot 10^{-4}$
10^3	$7.5 \cdot 10^{-4}$
10^4	$1.7 \cdot 10^{-3}$
10^5	$5.4 \cdot 10^{-3}$
10^6	$2.8 \cdot 10^{-3}$



The future: n_TOF vertical flight path at 20 m



Main beam characteristics at CERN nTOF

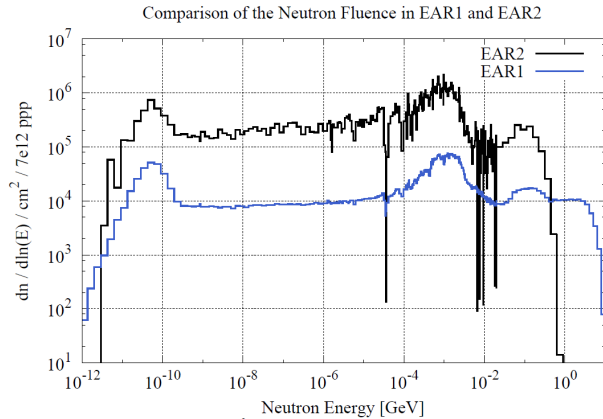


Fig. 8: Simulated neutron fluence per cm^2 in the existing n_TOF experimental area (EAR-1, blue line) and in the proposed facility above the n_TOF target (EAR-2, black line). It is worth noting that, while the neutron spectrum extends up to several GeV for the EAR-1, there is a sharp cut at ~ 300 MeV in EAR-2.

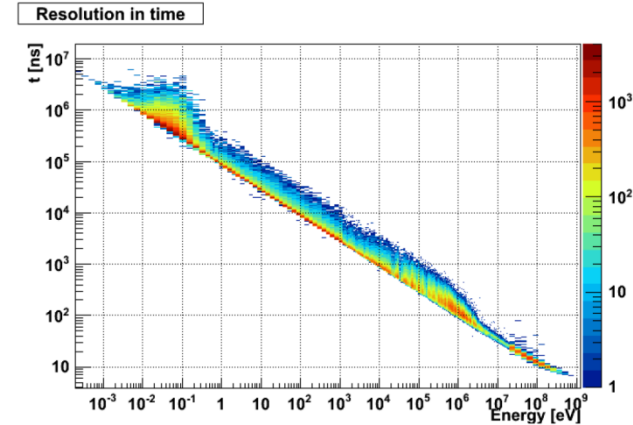


Fig. 9: Resolution in the EAR-2, showing the relation between time of arrival and neutron energy.

neutron intensity/7 10^{12} protons factor 25

time-of-flight energy correlation

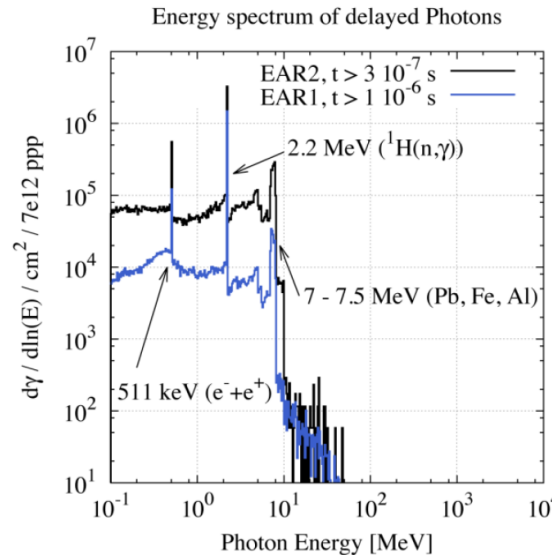
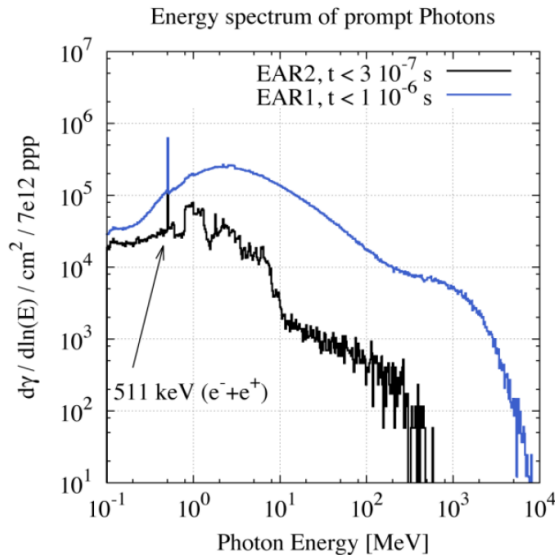
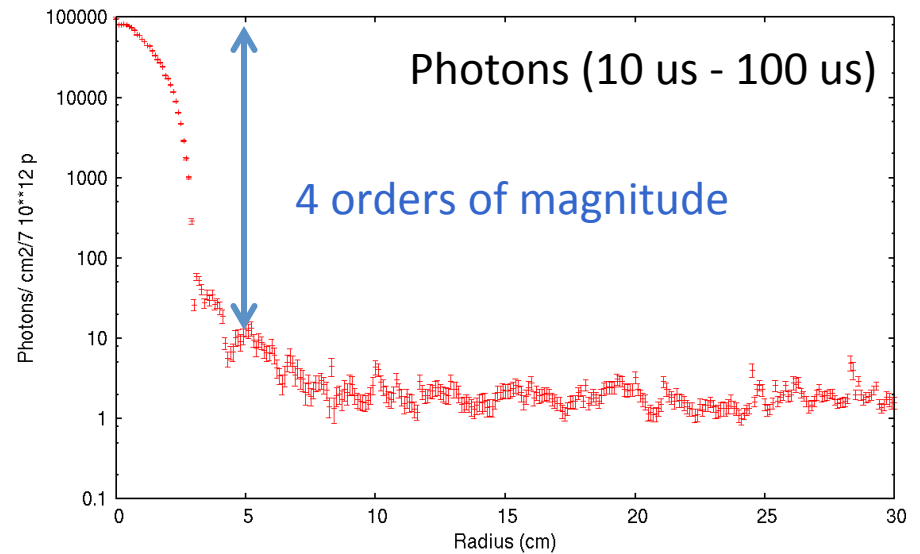
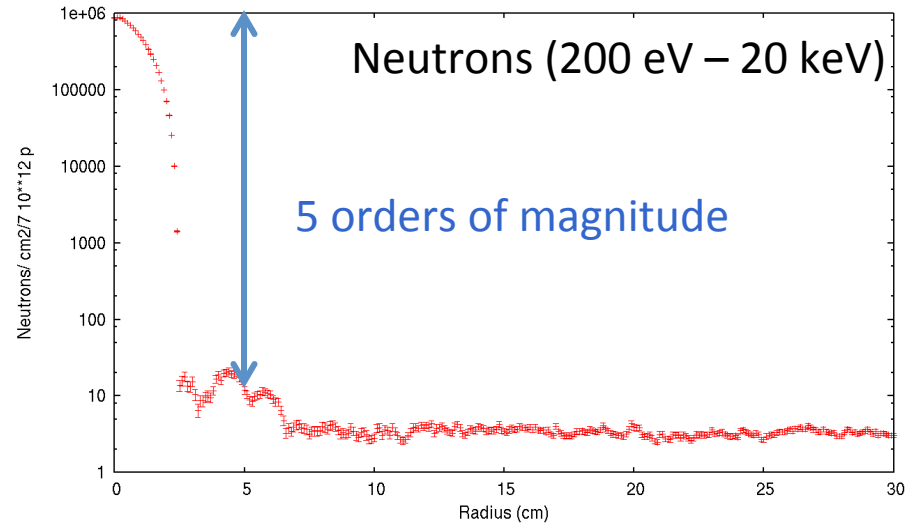
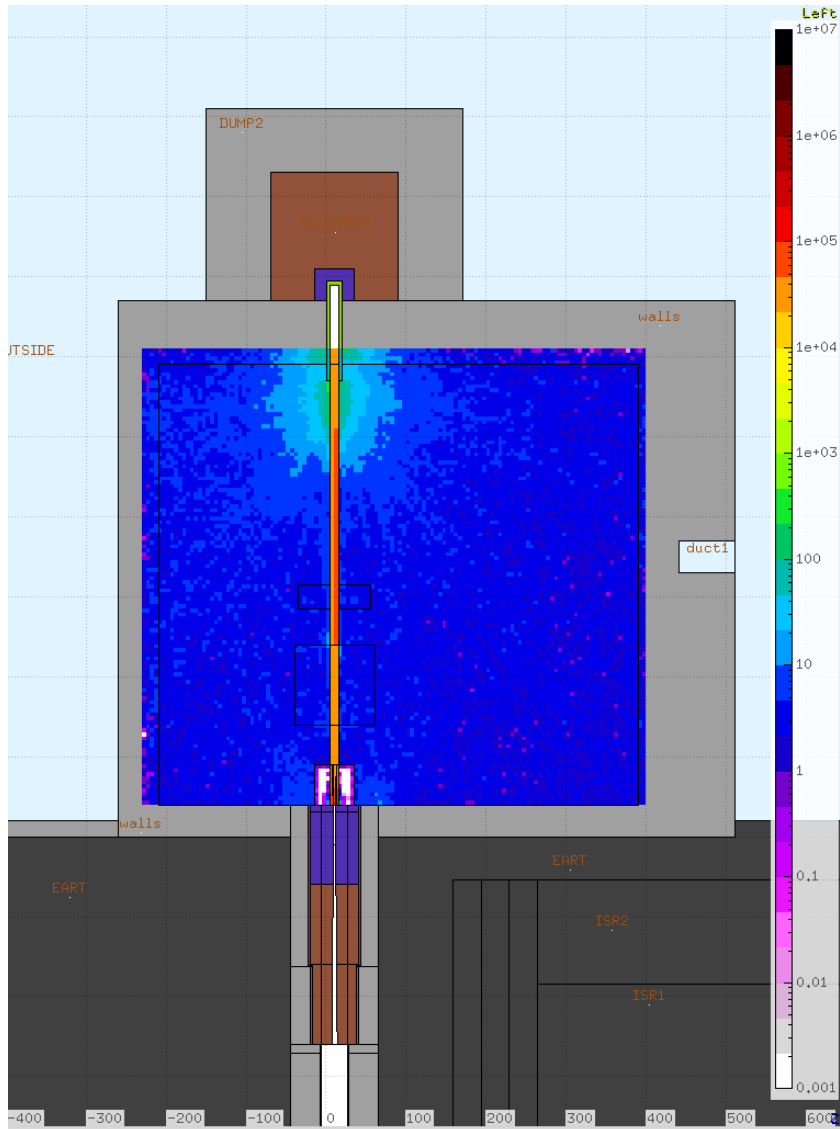


Fig. 12: Photon energy spectrum of prompt (left) and delayed (right) photons in both facilities.

Gammaflash reduced at EAR2

strong background from delayed photons (moderator, capture)

Detailed FLUKA simulation for the design of collimators and dump



Conclusions and perspectives

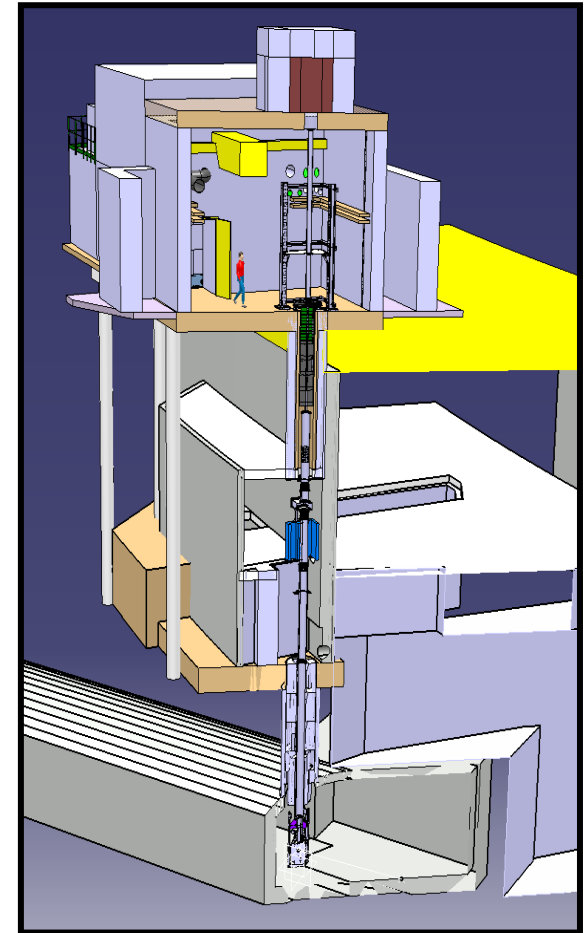
The new n_TOF EAR-2 20 m neutron beam line will be operative at CERN from July 2014

25 times higher flux (n/pulse) than n_TOF EAR1 (185 m)
250 times higher flux neutron rate (n/s) than n_TOF EAR1

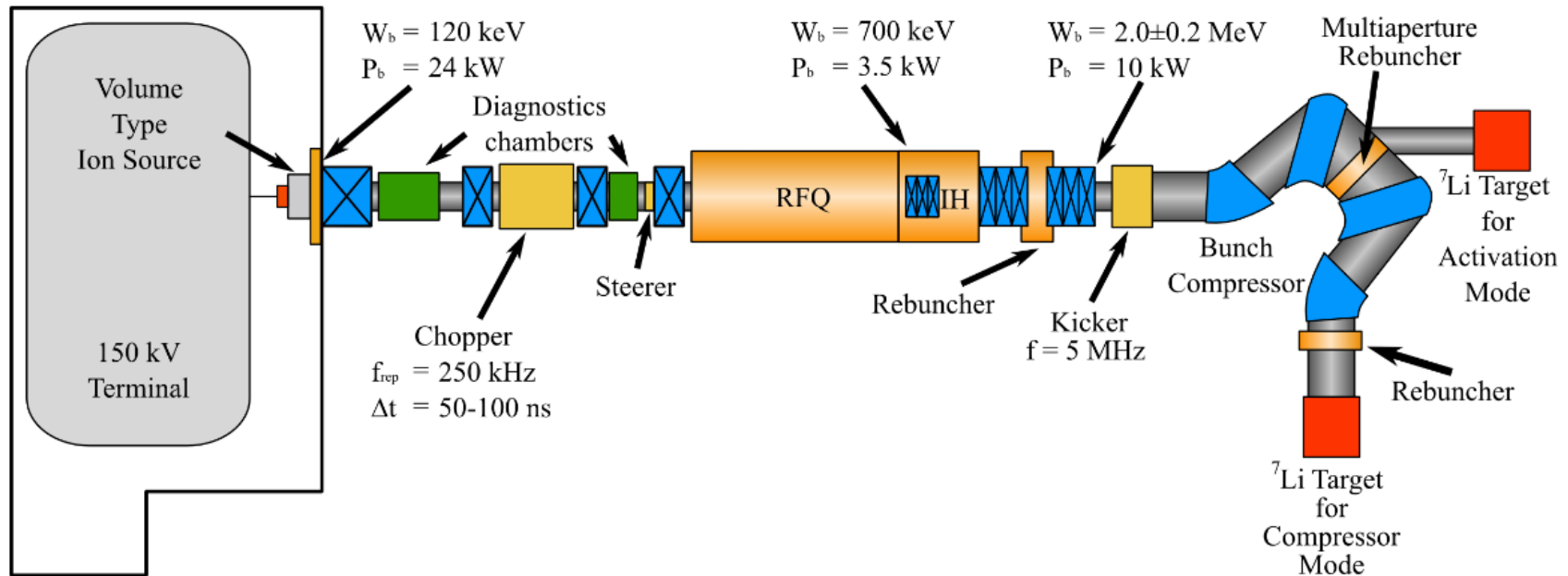
Reduced energy resolution (no RR above ~ 10 keV)
Runs in parallel to EAR1

First physics experiments by end 2014:

- Capture on fissile isotopes
- Capture on small mass s-process branching points
- Fission spectroscopy and prompt γ -rays with STEFF
- Elastic/inelastic reactions (HPGe or CsI+Si telescopes)
- Fission on high activity samples (e.g. ^{240}Pu)
- Irradiation of electronic components (@1.5 m)
- ...



Frankfurt Neutron source in the Stern-Gerlach Center (FRANZ) ${}^7\text{Li}(p,n){}^7\text{Be}$

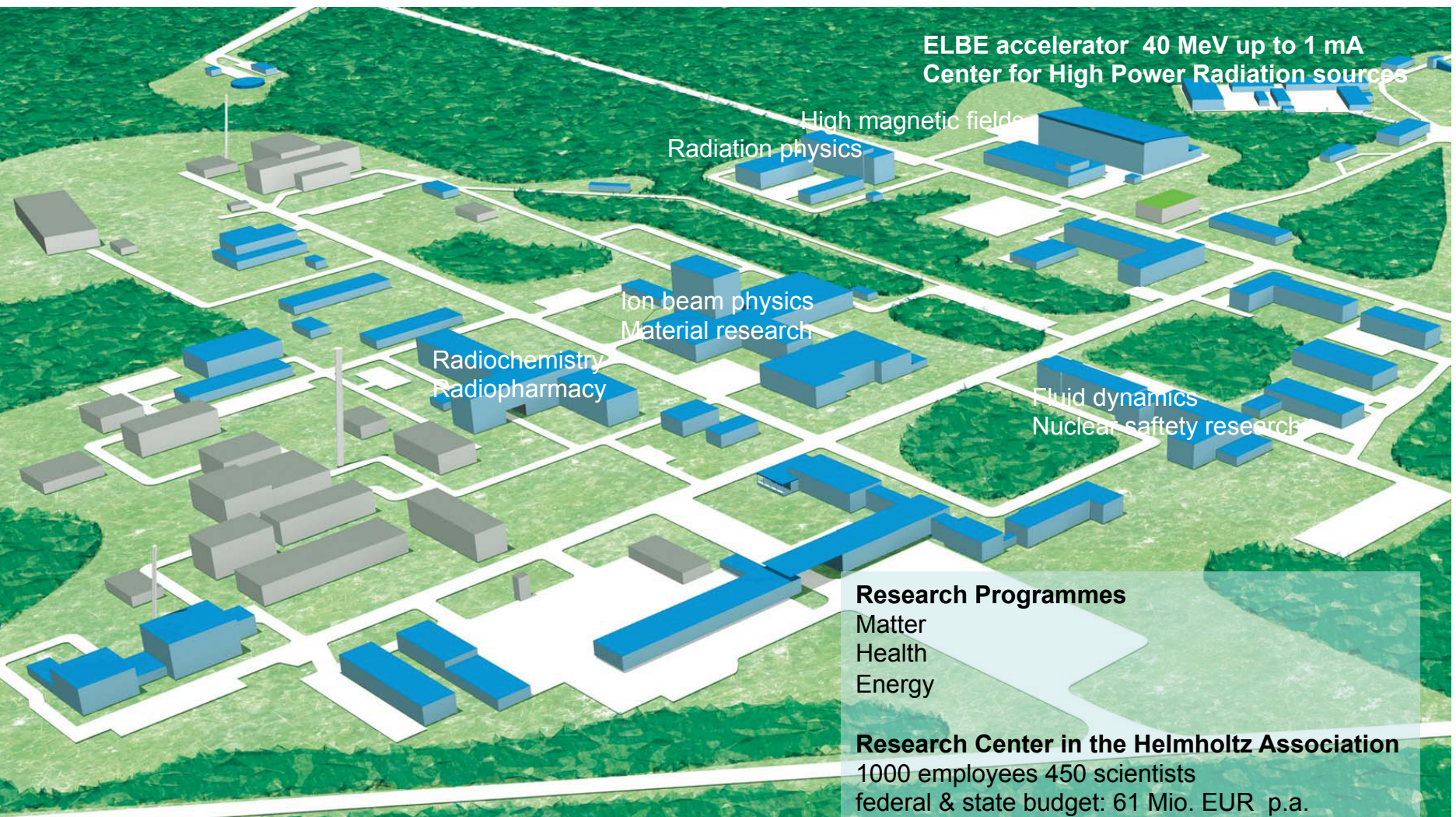


Proton accelerator: $E_p = 1.8 - 2.2 \text{ MeV}$, 20 – 250 kHz repetition rate,
Pulse length on target: 1 ns (bunch compression)
beam power 21 kW (average beam current = 10 mA)

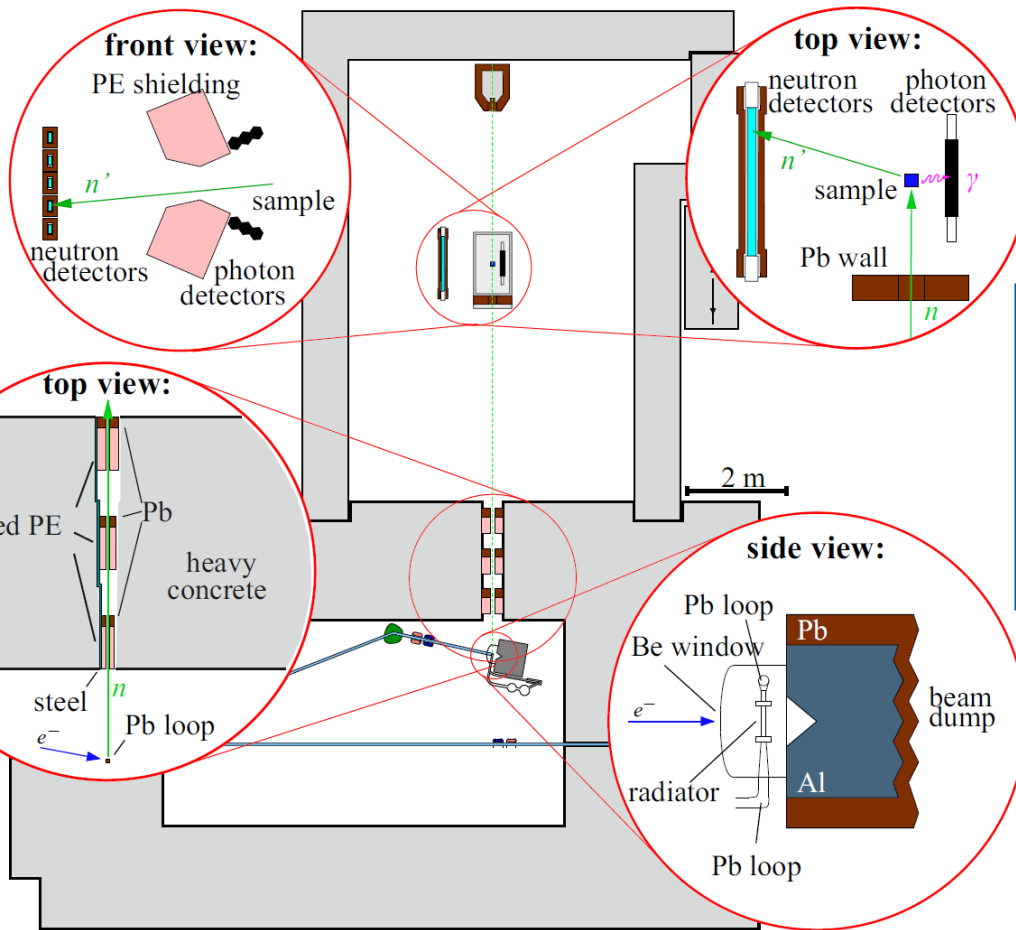
Neutron energy: 100 – 500 keV
Neutron intensity : $5 \cdot 10^{10} \text{ n/s}$
on target: $3 \cdot 10^7 \text{ n/s}$ flight path < 100 cm

→ Presentation by René Reifarh

Helmholtz-Zentrum Dresden-Rossendorf



Photoneutron Source nELBE



Characteristic parameters:

- repetition rate: 101 or 202 kHz
- flight path: 5 - 11 m
- source strength: ca. $1.6 \cdot 10^{11}$ n/s
- intensity @ target: ca. $2.5 \cdot 10^4$ n/cm²s
- energy range: 10 keV - 10 MeV
- energy resolution: < 1 %

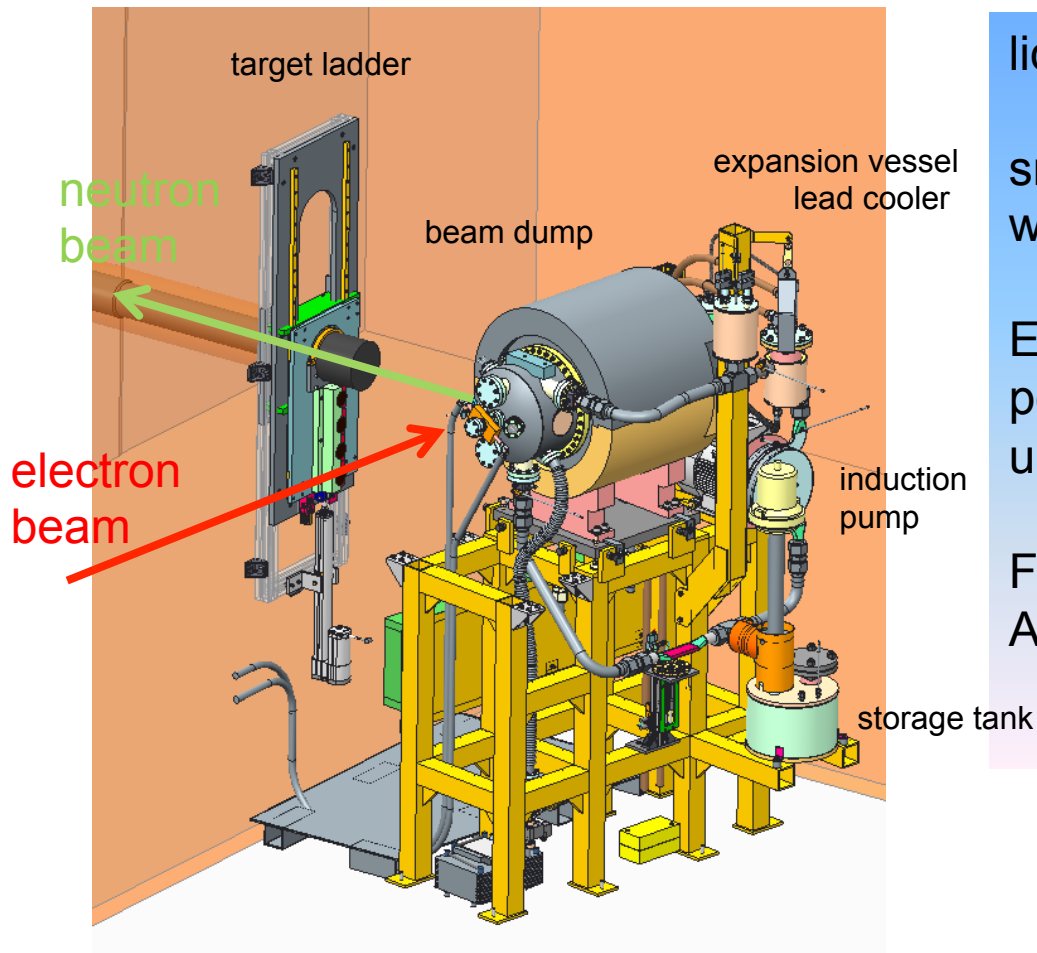
The only photo neutron source at a superconducting accelerator
New time-of-flight facility operational since August 2013.

Floor plan of the new nELBE neutron source and low scattering experimental hall.

The nELBE Neutron Time of Flight Facility
A.R. Junghans, et al.,
[J. Korean Phys. Soc. 59, 1593-1596 \(2011\)](#)
Proc. Int.Conf. Nucl. Data Science and Tech, 2013
New York



Liquid-Pb loop as neutron producing target



CAD design: Armin Winter

[E. Altstadt et al., Ann. Nucl. Energy 34 \(2007\) 36](#)

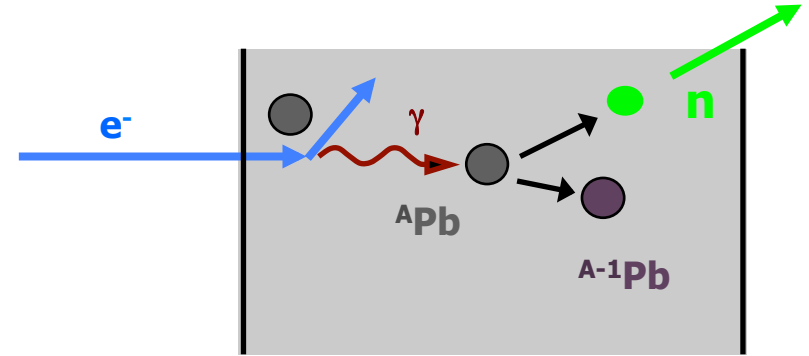
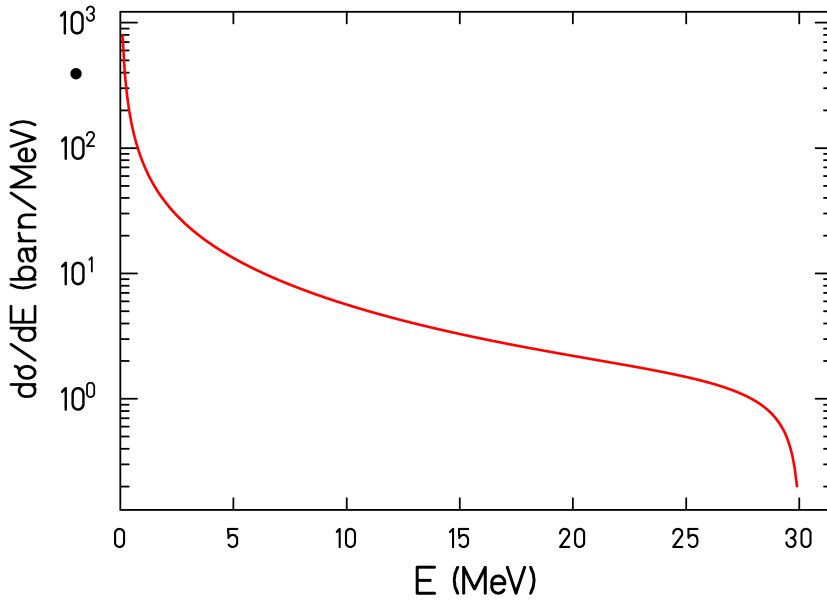
liquid lead circuit for heat transport

small Mo tube (11 mm diam.)
with liquid lead as neutron radiator

Electron beam power up to 40 kW
power density in the neutron radiator
up to 25 kW/cm^3

First beam with new Pb-loop:
August 30, 2013

Photoproduction of neutrons with bremsstrahlung



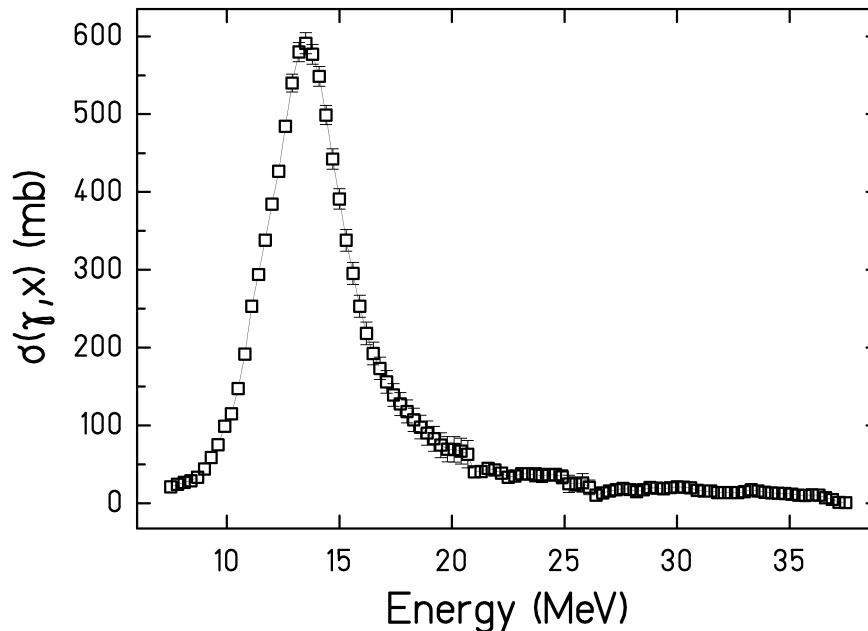
Bremsstrahlungsspectrum →
 Photonuclear excitation of
 Pb through the GDR
 Giant Dipole Resonance

Neutron production by
 (γ, xn) reactions

nELBE yield: $3 \cdot 10^{11}$ n/s with
 30 MeV 15 μA (Target: Pb, liquid)
 200 kHz

GELINA yield: $3 \cdot 10^{13}$ n/s with
 100 MeV 96 μA (Target: U (Hg cooled))
 800 Hz

Veyssiere et al., NPA 159 (1970) 561



Side remark: Normalization of photoneutron cross section measurements

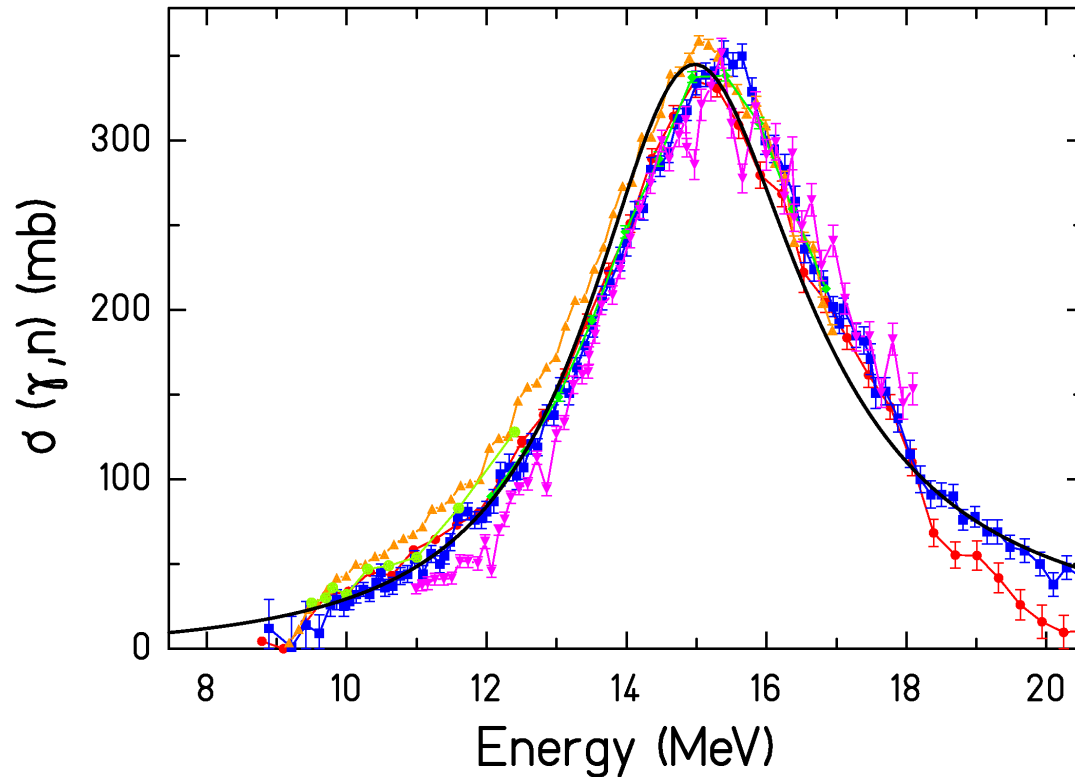


TABLE VI. Recommended normalization factors.

Isotope	Laboratory	Reference	Normalization factor
^{nat} Rb	Saclay	9	0.85±0.03
^{nat} Sr	Saclay	9	0.85±0.03
⁸⁹ Y	Saclay	9	0.82
⁸⁹ Y	Livermore	8	1.0
⁹⁰ Zr	Saclay	9	0.88
⁹⁰ Zr	Livermore	8	1.0
⁹¹ Zr	Livermore	8	1.0
⁹² Zr	Livermore	8	1.0
⁹³ Nb	Saclay	9	0.85±0.03
⁹⁴ Zr	Livermore	8	1.0
¹²⁷ I	Saclay	10	0.80
¹²⁷ I	Livermore	2	a
¹⁹⁷ Au	Saclay	12	0.93
¹⁹⁷ Au	Livermore	13	a
²⁰⁶ Pb	Livermore	11	1.22
²⁰⁷ Pb	Livermore	11	1.22
²⁰⁸ Pb	Livermore	11	1.22
²⁰⁸ Pb	Saclay	12	0.93
²⁰⁸ Bi	Livermore	11	1.22

^aDo not use.

Intercalibration of different photoneutron experiments.
Berman et al. Phys. Rev. C 36 (1987) 1286

➔ Renormalisation factors

This is not included in the Dietrich & Berman
GDR Atlas and RIPL2/3

Nuclear reactions in the statistical model

$$W_{if} = \frac{2\pi}{\hbar} |\bar{H}_{fi}|^2 \rho_f$$

$$W_{fi} = \frac{2\pi}{\hbar} |\bar{H}_{if}|^2 \rho_i$$

Fermi's golden rule:

Averaged Matrix elements

Matrix elements have random phases because of many degrees of freedom

Averaging over $\Delta E > \Gamma$

$$\rho_c W_{c\beta} = \rho_\beta W_{\beta c} \quad \text{for hermitian operators } |H_{if}|^2 = |H_{fi}|^2$$

$$\rho_\beta = \rho_b^{frei} \cdot \rho_B(U)$$

$$\rho_b^{frei} = \frac{\tau}{2\pi^2 \hbar^3} m_\beta p_\beta$$

$$p_\beta = p_b = -p_B$$

$$\frac{d\sigma}{d\Omega} = \frac{W_\tau}{4\pi v_i}$$

$$\sigma = \frac{W_\tau}{v_i}$$

$$\sigma_{\beta c} = \frac{W_{\beta c} \tau}{v_\beta}$$

Cross section for CN formation in channel β

$$W_{\beta c} = \frac{v_\beta \sigma_{\beta c}}{\tau}$$

$$\rho_c(E) W_{c\beta} = \frac{v_\beta \sigma_{\beta c}}{2\pi^2 \hbar^3} m_\beta p_\beta \rho_B(U) \frac{v_\beta \sigma_{\beta c}}{\tau}$$

$$U = E - \epsilon - S_n$$

$$\epsilon_\beta = \frac{1}{2} v_\beta p_\beta = \frac{1}{2} m_\beta v_\beta^2$$

$$W_{c\beta}(\epsilon_\beta) = \frac{\rho_B(U)}{\rho_c(E)} \frac{m_\beta \epsilon_\beta \sigma_{\beta c}(\epsilon_\beta)}{\pi^2 \hbar^3}$$

Reaction rate of the CN decay in channel β as a function of kinetic energy

Depends on the ratio of the **level densities** of the **final nucleus** and the **compound system**

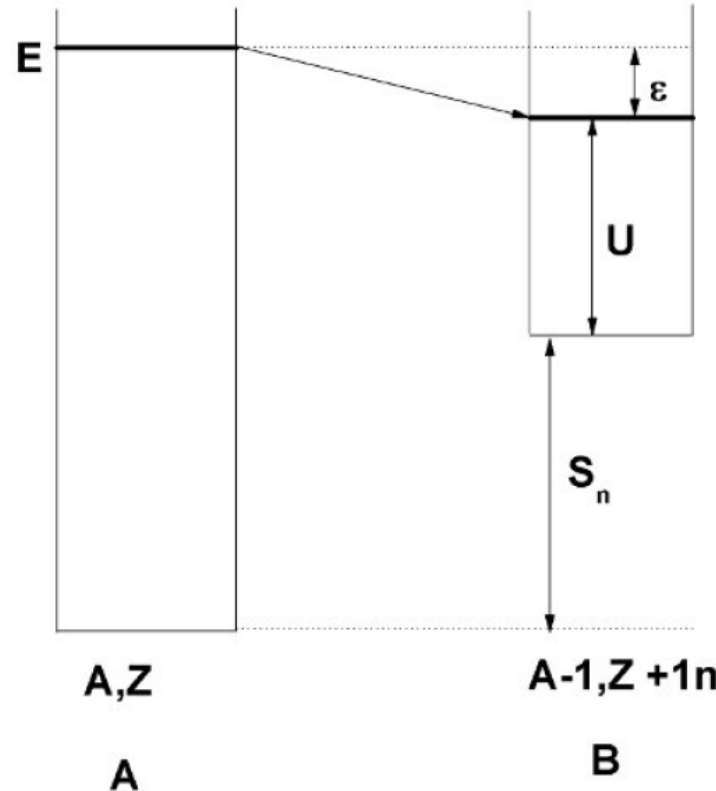
Evaporation spectra from CN decay

$$\rho_c \propto \frac{1}{T} e^{E/kT}$$

$$\rho_B \propto \frac{1}{T} e^{U/kT} = \frac{1}{T} e^{(E-\epsilon-S_n)/kT}$$

$$\frac{\rho_B(U)}{\rho_c(E)} = \text{const} \cdot e^{-\epsilon/kT}$$

$$W(\epsilon_\beta) = \text{const} \cdot \sigma_{\beta c} \epsilon_\beta e^{-\epsilon/kT}$$



$$U = E - \epsilon - S_n$$

At very large ϵ
Transitions to
Discrete excited
States of the
Residual nucleus
B

The emission spectrum depends on:

The **level density** of the compound nucleus ρ_c

The **level density** of the residual nucleus ρ_B

And the inverse cross section of compound nucleus formation

For neutron emission $\sigma_{\beta c}$ is not strongly energy depend. \rightarrow Maxwellian energy spectrum

For charged particle emission: Transmission through the Coulomb-Barrier

Neutron evaporation spectra

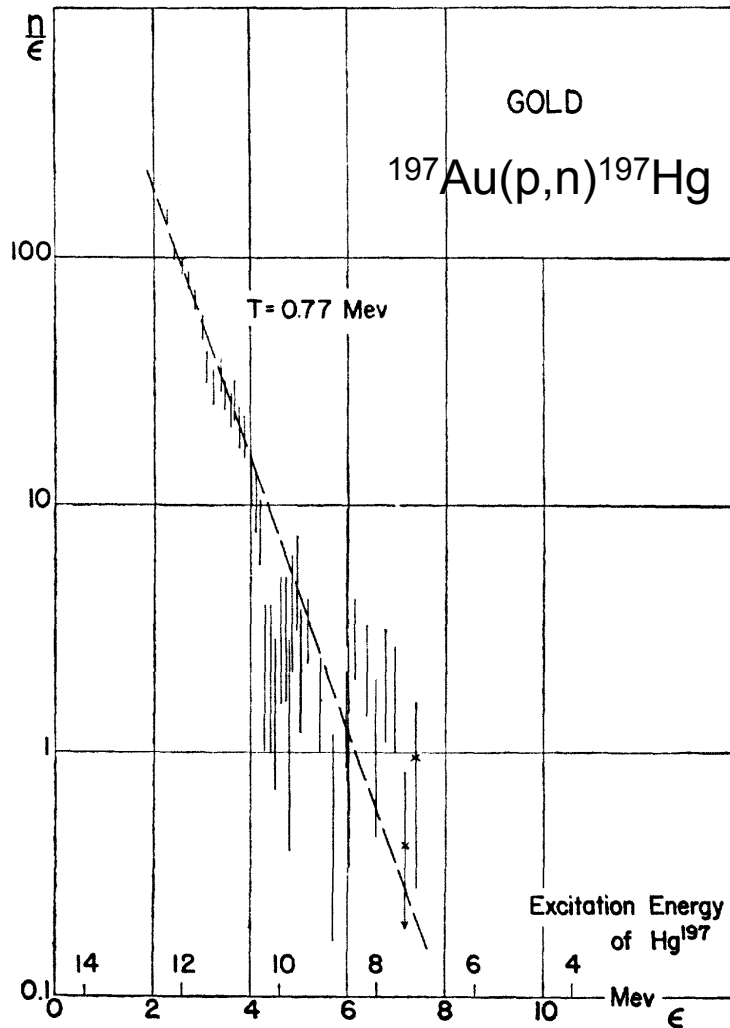


FIG. 11. Relative level density of Hg^{197} .

Gugelot, Phys. Rev. 81 (1951) 51

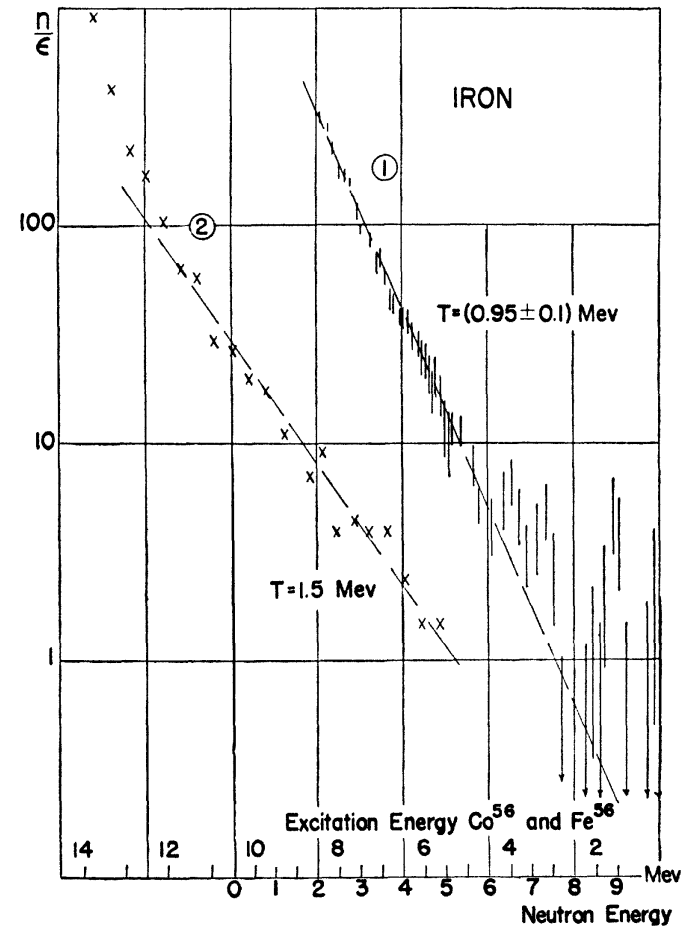
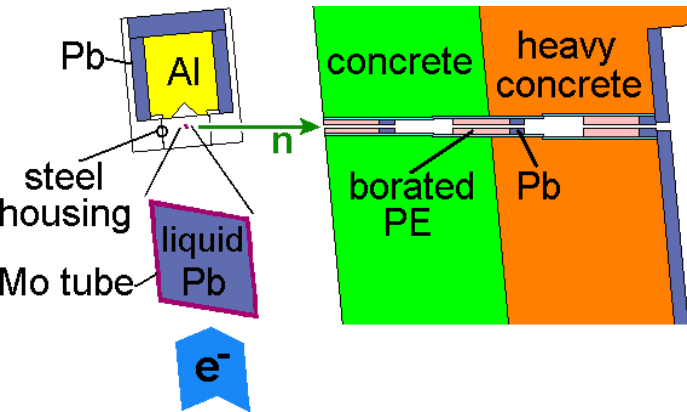


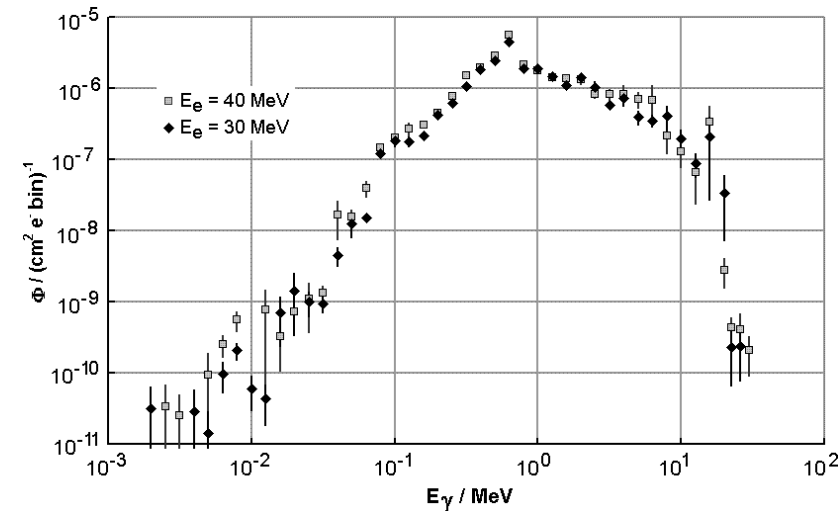
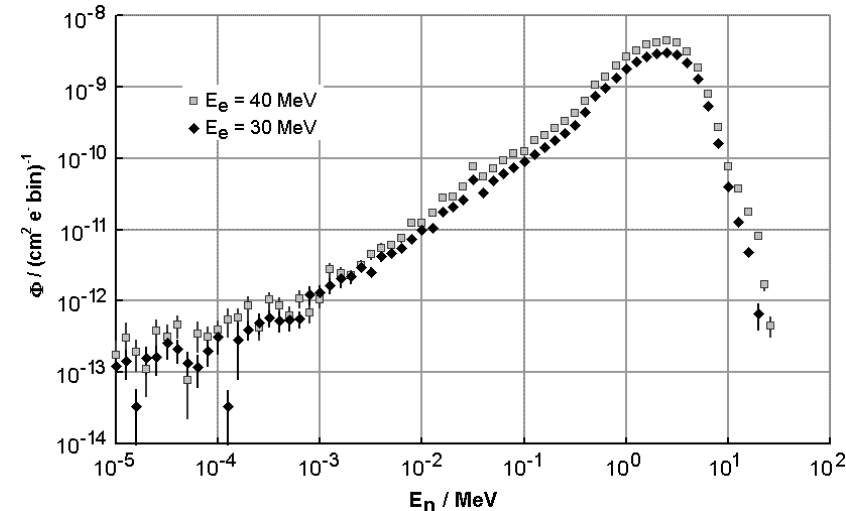
FIG. 9. Relative level density of Co^{56} and Fe^{56} . Curve 1: represents the relative level density for Co^{56} obtained from the neutron spectrum; curve 2: shows the relative level density of Fe^{56} as observed from the inelastic scattering of 16-Mev protons by iron (reference 38).

MCNP: Neutron and photon source spectra

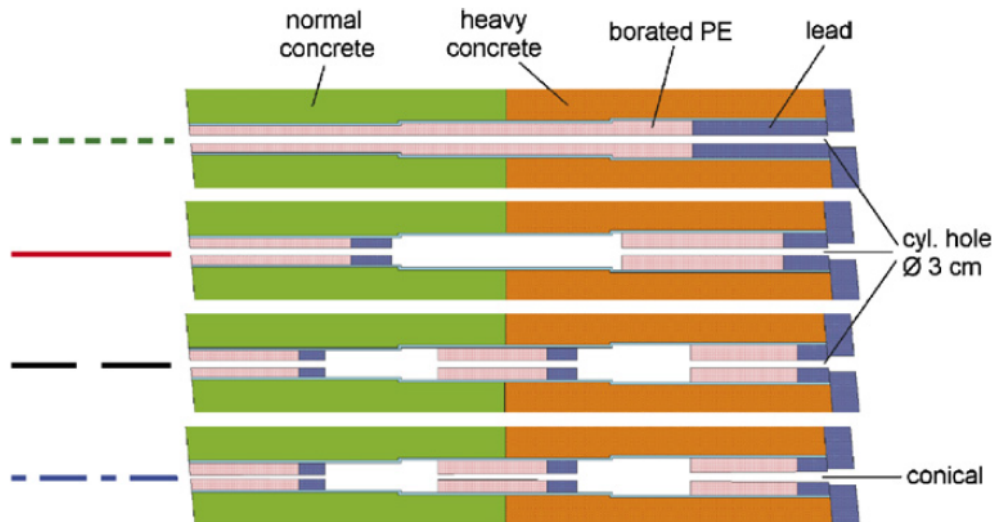


- *Mode e p n* calculation with photonuclear physics turned on
- Photonuclear cross sections for Pb and Mo adopted
- Electrons started uniformly outside Mo channel from circular disc, $\varnothing = \varnothing_{\text{beam}} = 8$ mm
- Neutron and photon source distributions detected in collimator direction
- Distributions used as source spectra in later simulations – n & γ started uniformly from a cylindric volume (= intersection between e^- beam and Mo/Pb radiator)

→ [J. Klug et al. NIM A 577 \(2007\) 641](#)



Collimator simulation for nELBE: neutrons



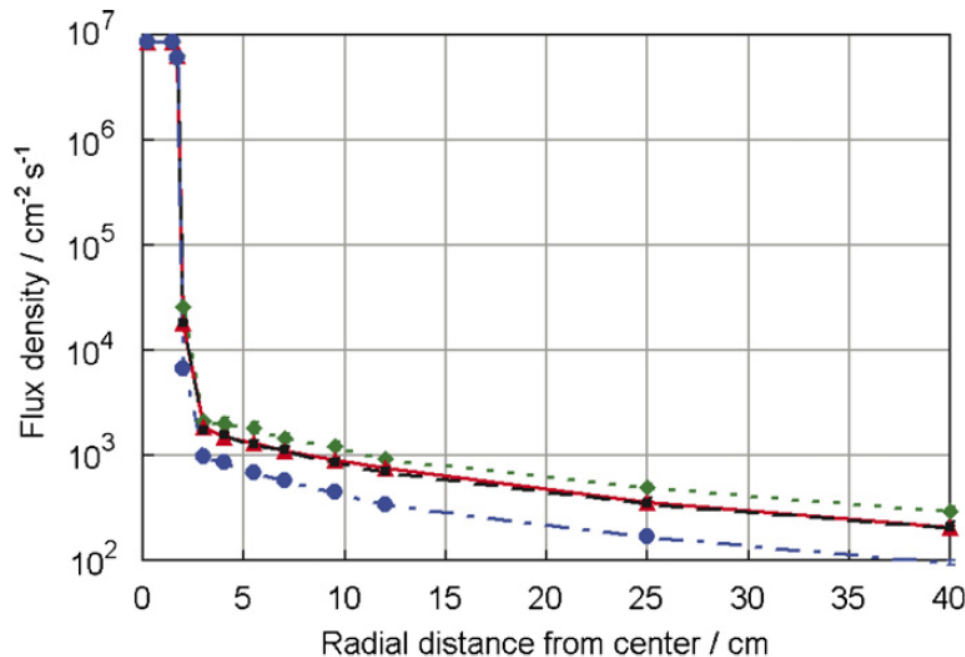
Collimator for neutrons with
Shielding capability for bremsstrahlung
from the photo neutron source

Principle:

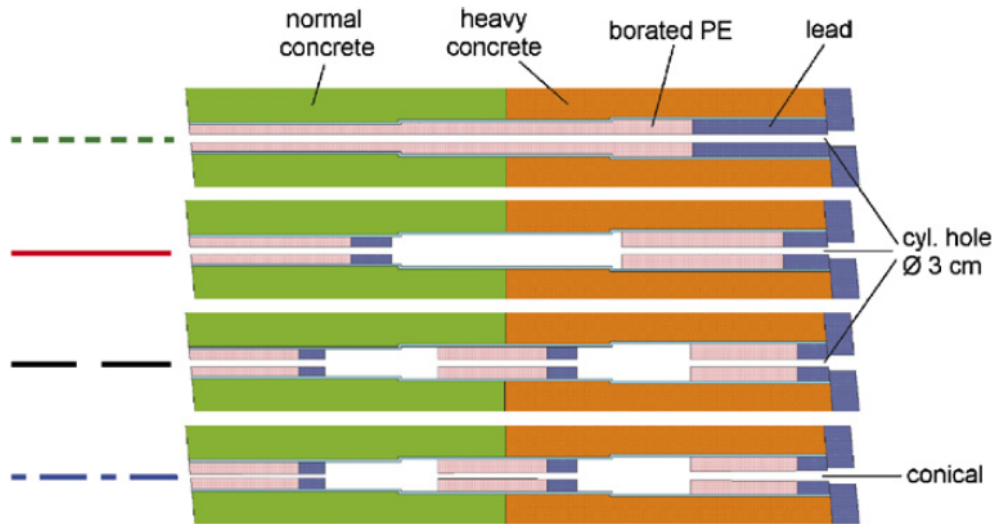
Pb for photon attenuation

borated PE for neutron attenuation

1. massive collimator has the largest beam halo
2. Two or three collimator segments do not make a big difference
3. A conical profile reduces the beam Halo compared with a cylindrical profile



Collimator simulation for nELBE: bremsstrahlung



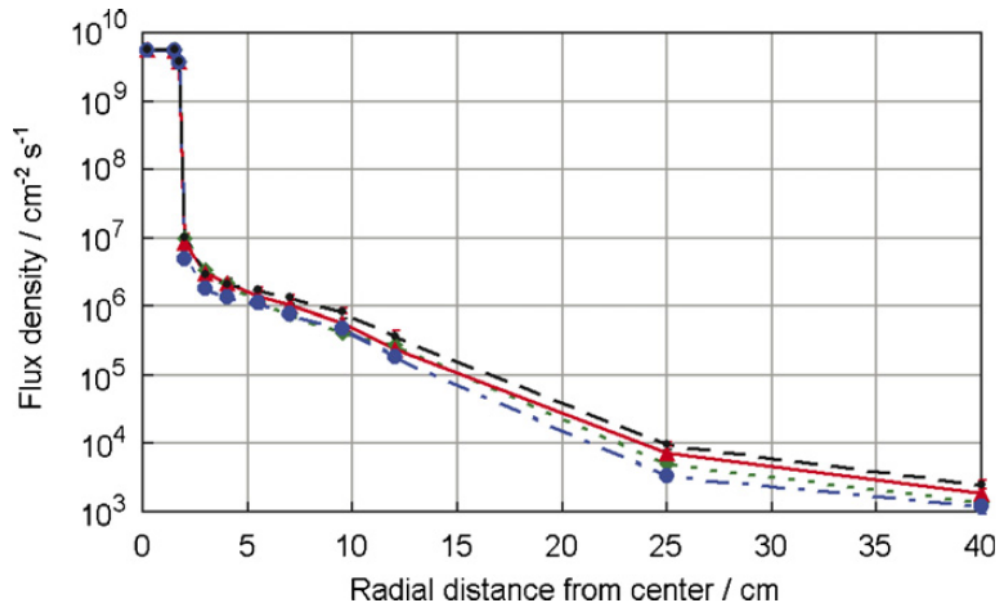
Collimator for neutrons with
Shielding capability for bremsstrahlung
from the photo neutron source

Principle:

Pb for photon attenuation

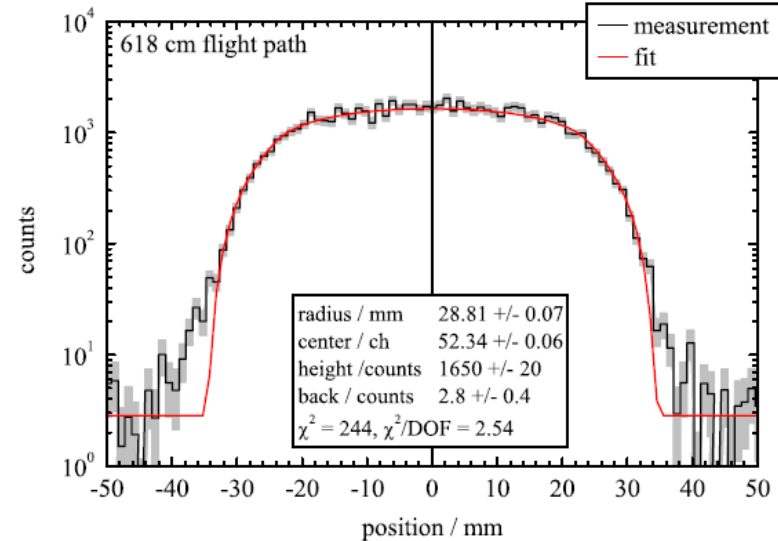
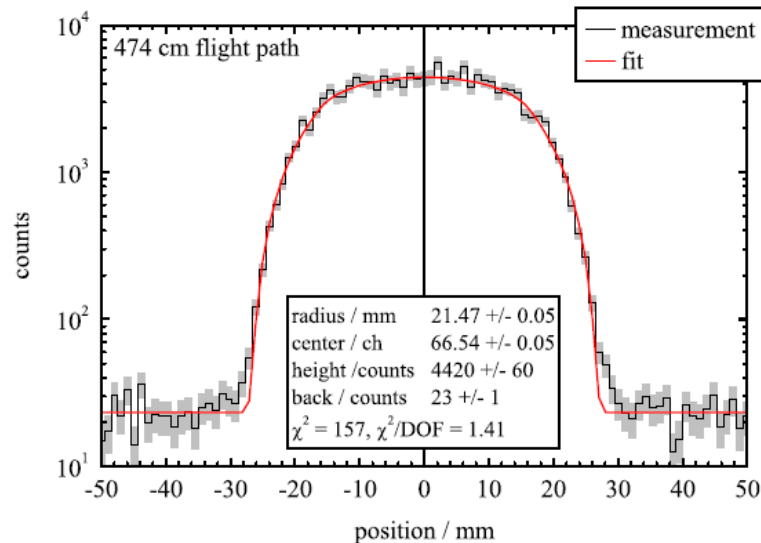
borated PE for neutron attenuation

1. Photon halo is much wider than the neutron halo
2. Three segments seem to be a little better than two segments
3. A conical profile reduces the beam halo compared with a cylindrical profile



Measured beam profile at nELBE

R. Beyer et al. / Nuclear Instruments and Methods in Physics Research A 723 (2013) 151–162

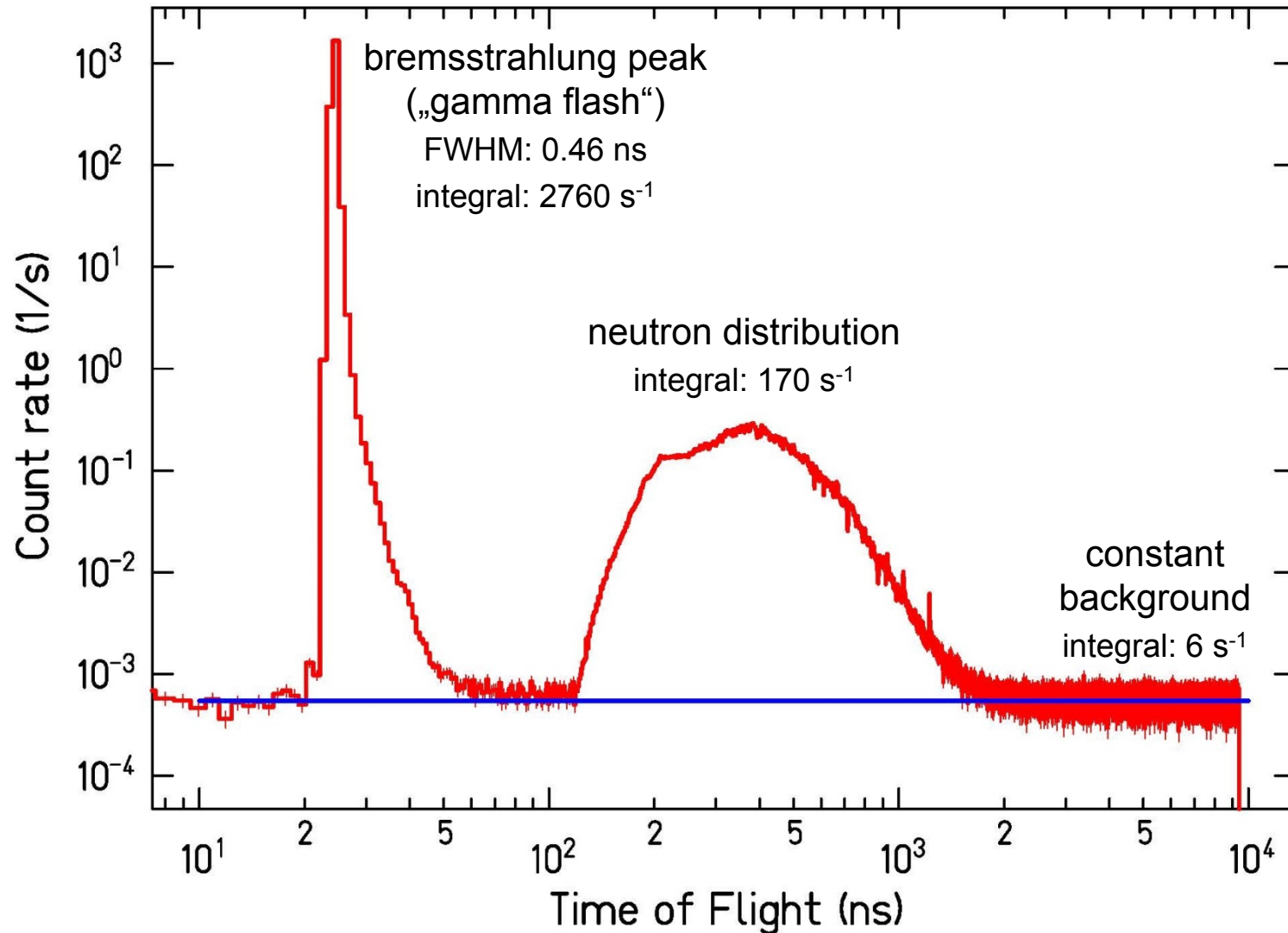


Beam profile from a one dimensional scan with a bar shaped scintillator
By time-of-flight only the neutron component is measured.
Reduction of the count rate in the beam halo by a factor $1.7 \cdot 10^{-3}$

The fitted curve assumes perfect rectangular beam profile with no halo.
The measurement position is farther away from the collimator exit.
Additional scattering by layers of matter in the beam (air, fission chamber)

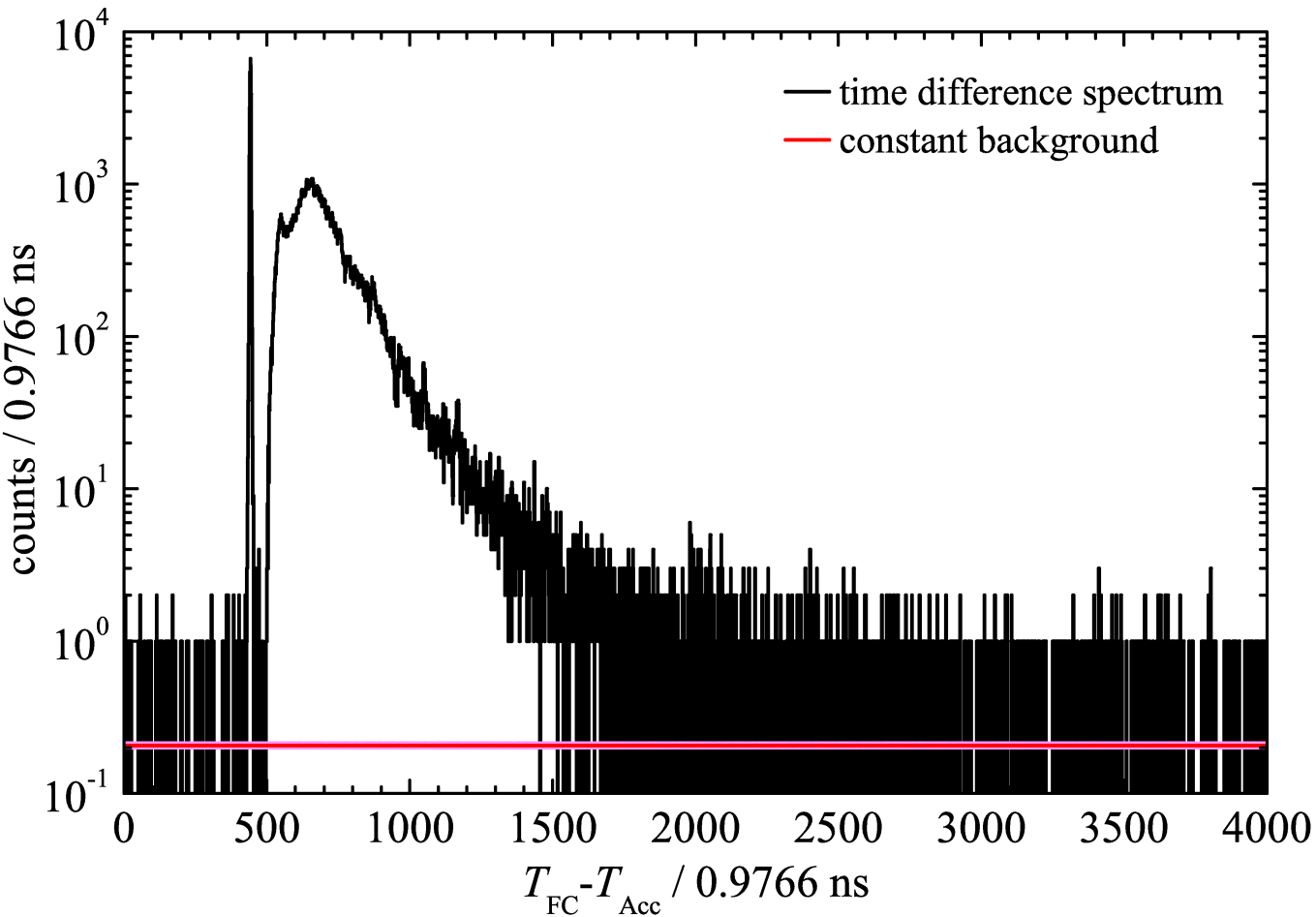
Time of flight spectrum

dead-time corrected count rate with ^{197}Au sample (red) and fitted background (blue)



neutron intensity strongly reduced for transmission measurement
detection threshold ca. 5 keV

nELBE time of flight spectrum

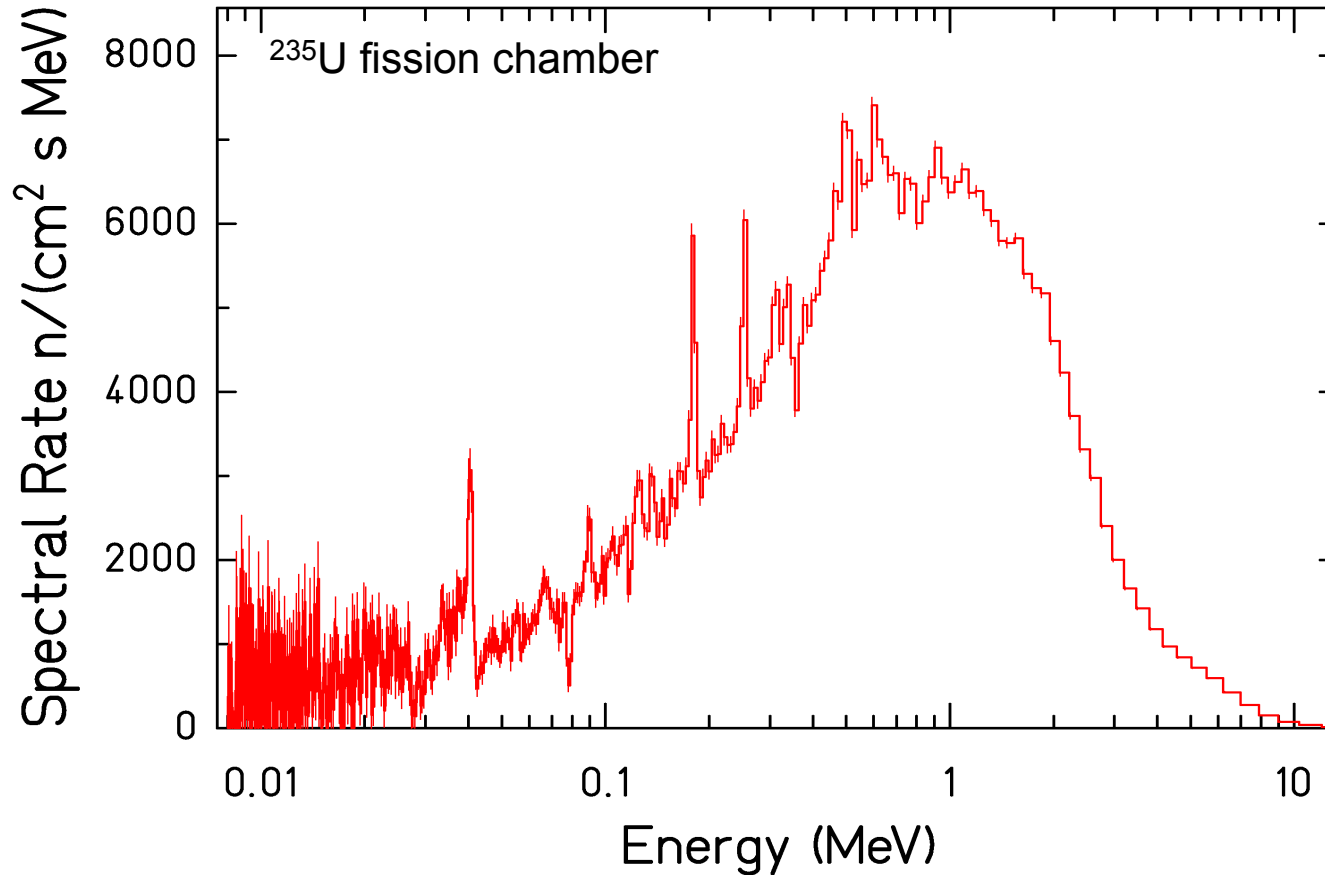


^{235}U fission chamber H19 from PTB.

Bremsstrahlung-induced fission is visible.

Time resolution from peak width = 3.8 ns FWHM

nELBE neutron spectrum



Measurement time : 49.4 h $I_{e^-} = 15 \mu\text{A}$, $E_{e^-} = 31 \text{ MeV}$

Flight path 618 cm

Absorption dips : 78, 117, 355, 528, 722, 820 keV ^{208}Pb scattering resonances

Emission peaks: 40, 89, 179, 254, 314, 605 keV near threshold photoneutron emission

In ^{208}Pb (strong capture resonances of ^{207}Pb)

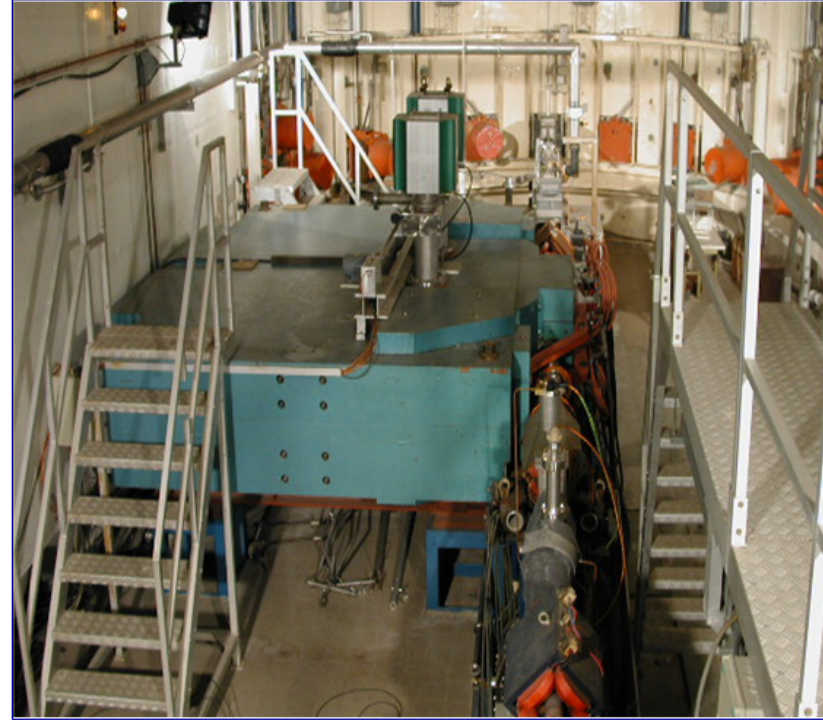
[R. Beyer et al., NIMA723 \(2013\) 151](#)

Electron accelerator



- 150 MeV electron accelerator
- 10 ns burst, 10 A peak
- 800 bursts/s

Compression magnet



- Pulse compression magnet
- <1 ns burst, >100 A peak

Electron-neutron conversion target



- Uranium target – rotating, mercury cooled
- $4 \cdot 10^{10}$ neutrons / burst

water-filled Be moderators

12 Flight paths
8 to 400 m



moderated or fast neutron spectrum
24 h/d, 100 h/w

GELINA target moderator / tof dependent background

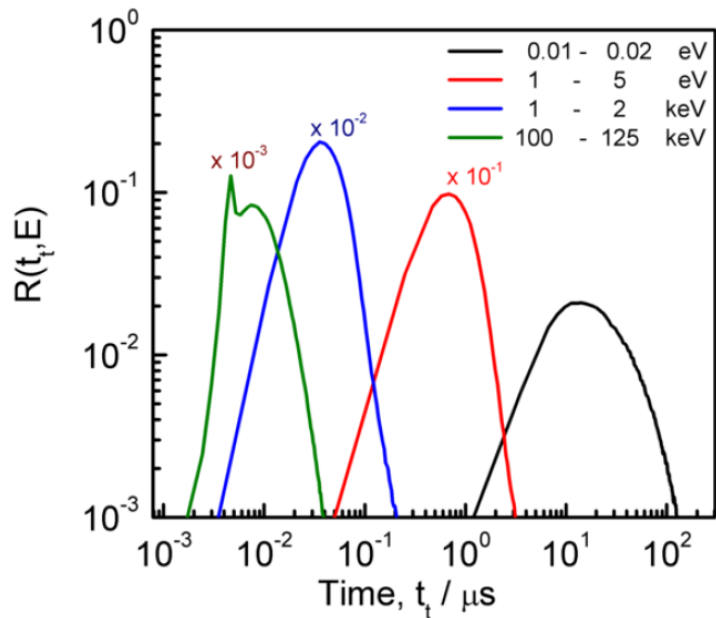


FIG. 1. The probability distribution of the time t_t that a neutron spends in the target-moderator assembly of GELINA.

slowing down time longer for lower neutron energy

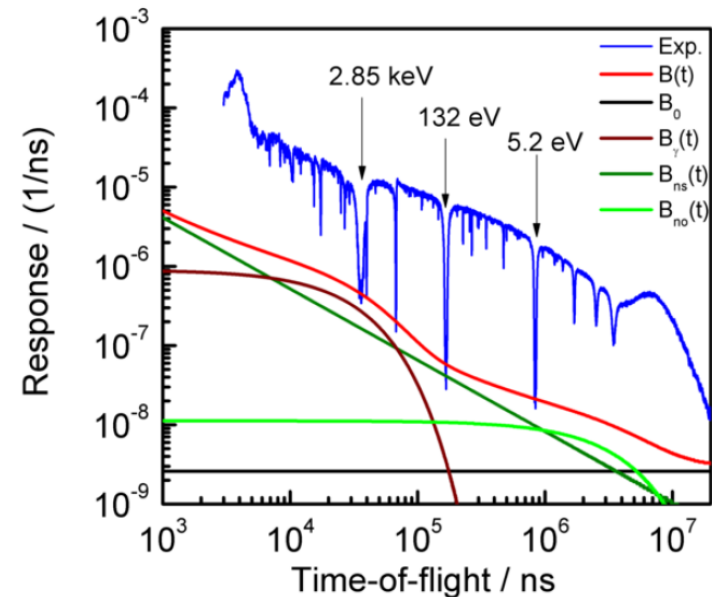


FIG. 11. The response of a Li-glass detector as a function of TOF for measurements at GELINA, is shown together with the total background and the contribution of the different components. The response is the result of a sample-in measurement on ^{241}Am [69].

tof dependent background:
capture γ rays in hydrogen moderator
overlap neutron, scattered neutrons and
neutrons from adjacent beamlines

GELINA neutron spectrum and energy resolution

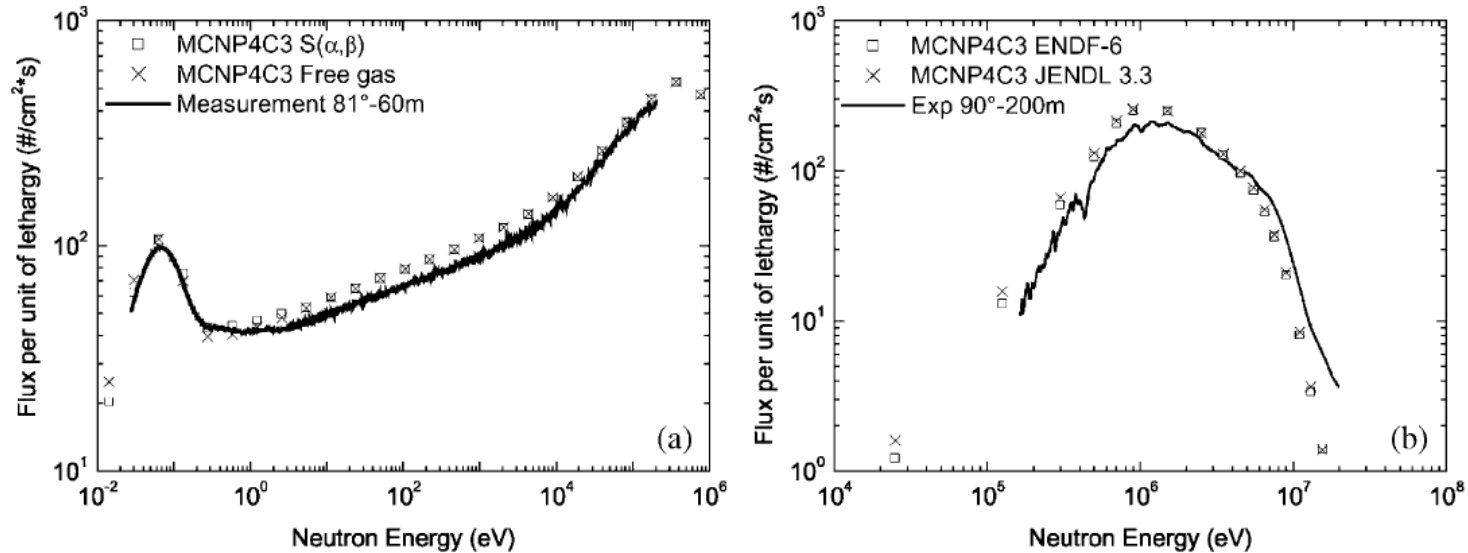
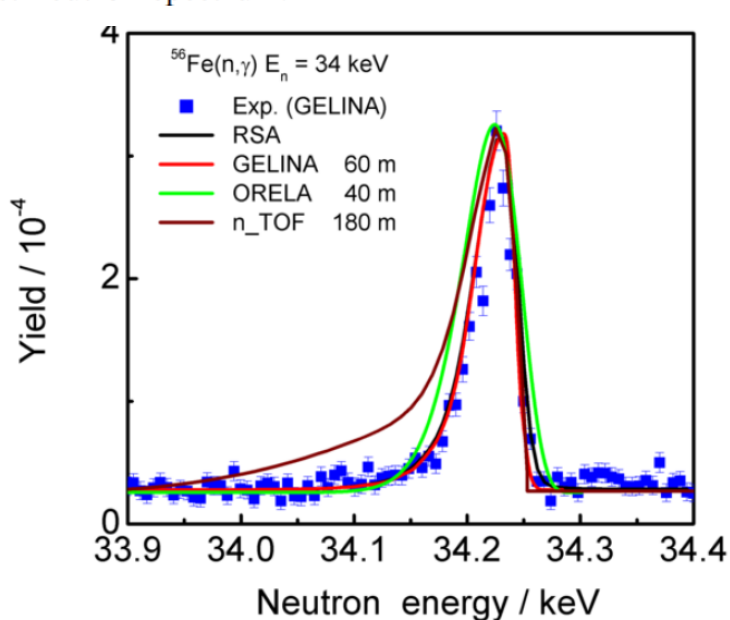


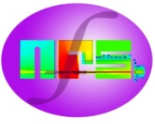
Fig. 4. Neutron flux per unit of lethargy in the flight-path. (a) 81°—60 m of the moderated neutron spectrum; (b) 90°—200 m of the fast neutron spectrum.



Fast neutron spectrum from 0.1 – 18 MeV

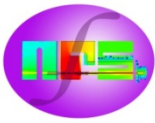
GELINA:

- width is dominated by the tof-resolution
resonance total $\Gamma \approx 2$ eV
Doppler width (FWHM) ≈ 13 eV
ToF resolution (FWHM) ≈ 40 eV
- photoneutron sources tend to have a higher resolution than spallation neutron sources (larger target-moderators required)

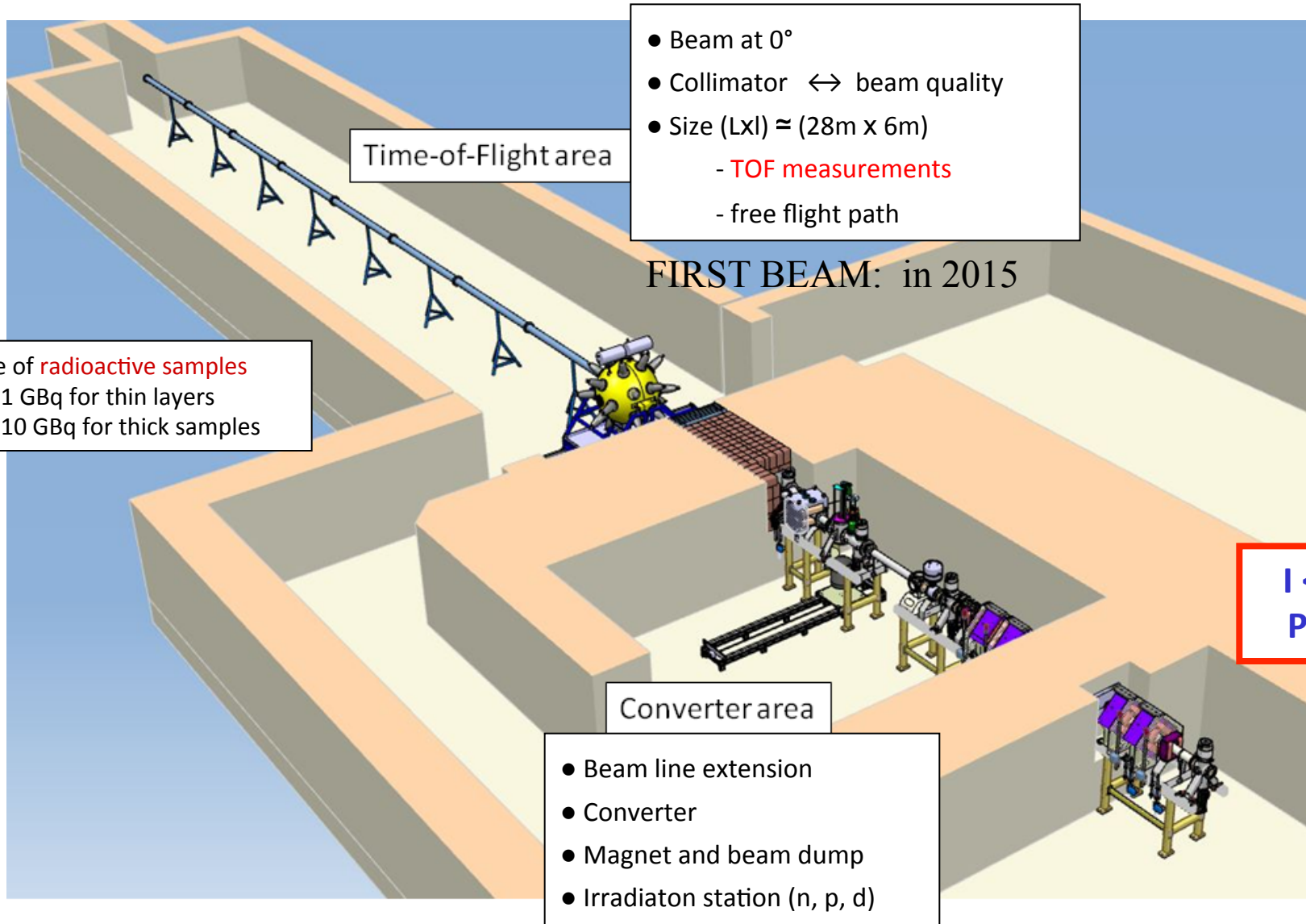


The Neutrons For Science facility at SPIRAL-2

X. Ledoux and the NFS collaboration



Description



- Beam at 0°
- Collimator ↔ beam quality
- Size (LxI) ≈ (28m x 6m)
 - TOF measurements
 - free flight path

Time-of-Flight area

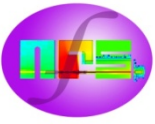
FIRST BEAM: in 2015

Use of radioactive samples
A < 1 GBq for thin layers
A < 10 GBq for thick samples

I < 50 μA
P < 2 kW

Converter area

- Beam line extension
- Converter
- Magnet and beam dump
- Irradiation station (n, p, d)



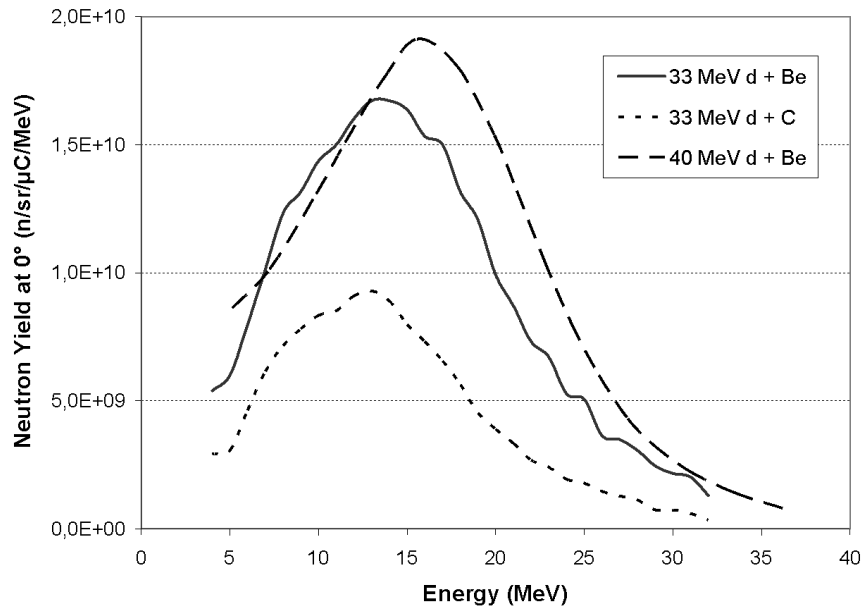
Neutron spectra provided at NFS

Characteristics of the beams the LINAG :

- 40 MeV deuteron and 33 MeV proton
- $I_{\max} = 5 \text{ mA}$
- Pulsed beam $F_0 = 88 \text{ MHz}$ $T = 11 \text{ ns}$ Burst width = 200 ps

Continuous spectrum :

$E_{\max} = 40 \text{ MeV}$, $\langle E \rangle = 14 \text{ MeV}$
thick converter (1cm)



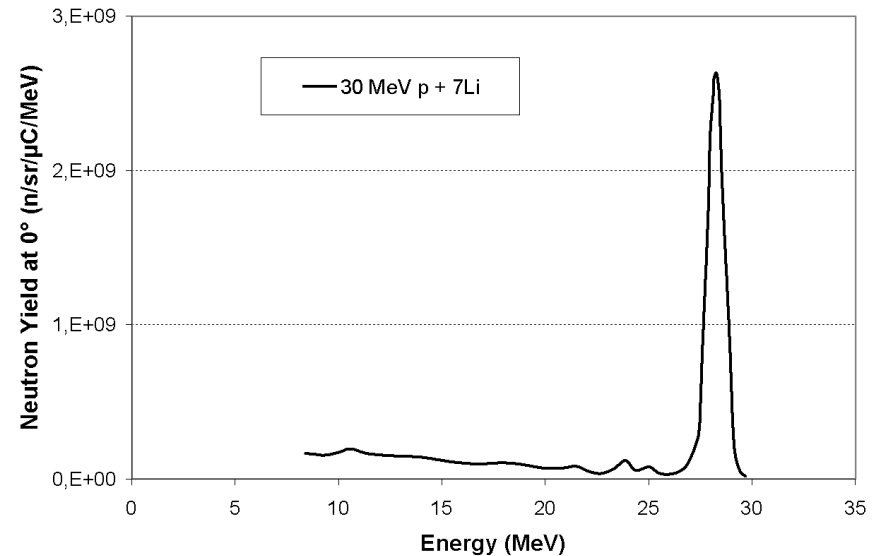
J. P. Meulders et al., Phys. Med. Biol. (1975)vol 20 n°2, p235

M. J. Saltmarsh et al., NIMA145 (1977) p81-90

⇒ **Similar to IFMIF spectrum**

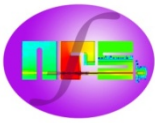
Quasi-monokinetic beam :

$E_n = \text{up to } 31 \text{ MeV}$
Thin converter (1-3 mm)



C. J. Batty et al., NIM 68 (1969) p273-276

Single bunch selector for time-of-flight measurements:
Repetition rate: 150 kHz – 1 MHz



Neutron flux in the TOF area

NFS : 40 MeV d + Be

WNR : Los Alamos

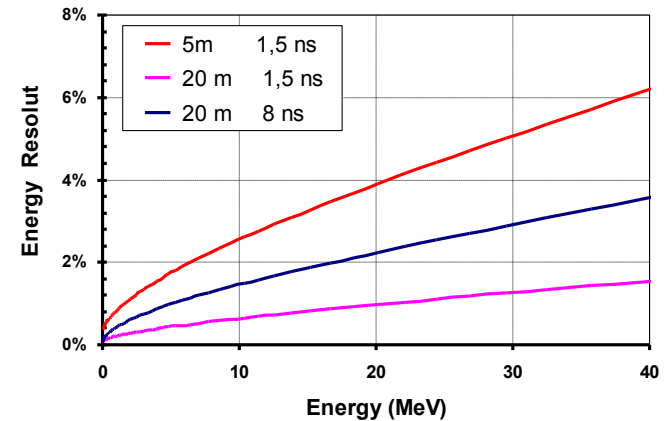
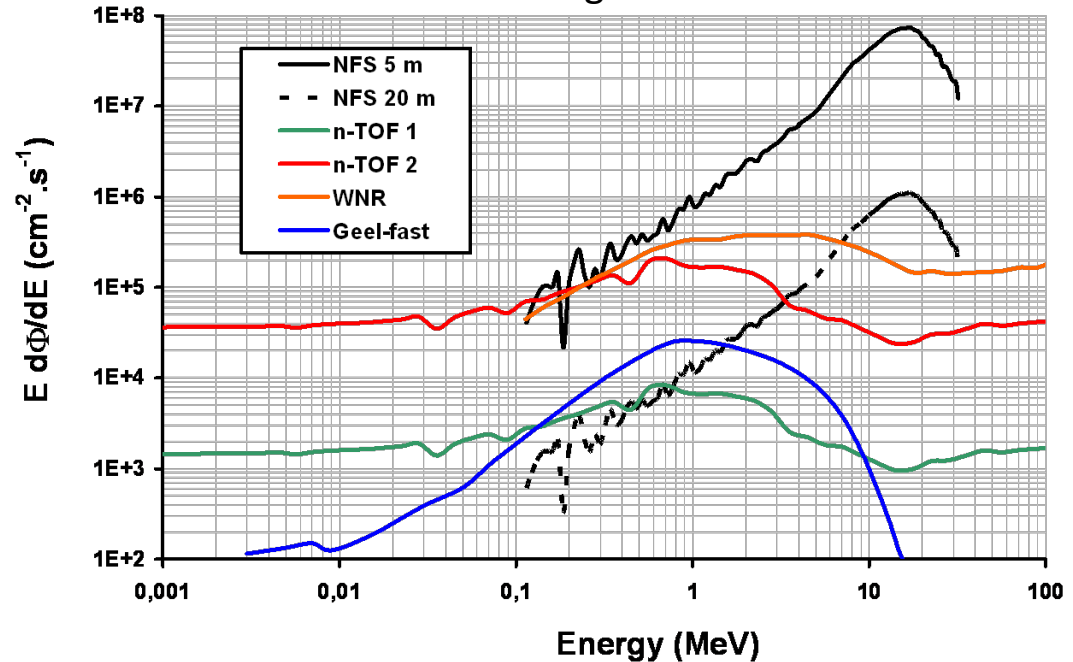
n-TOF 2 : CERN

n-TOF 1 : CERN

GELINA : Geel

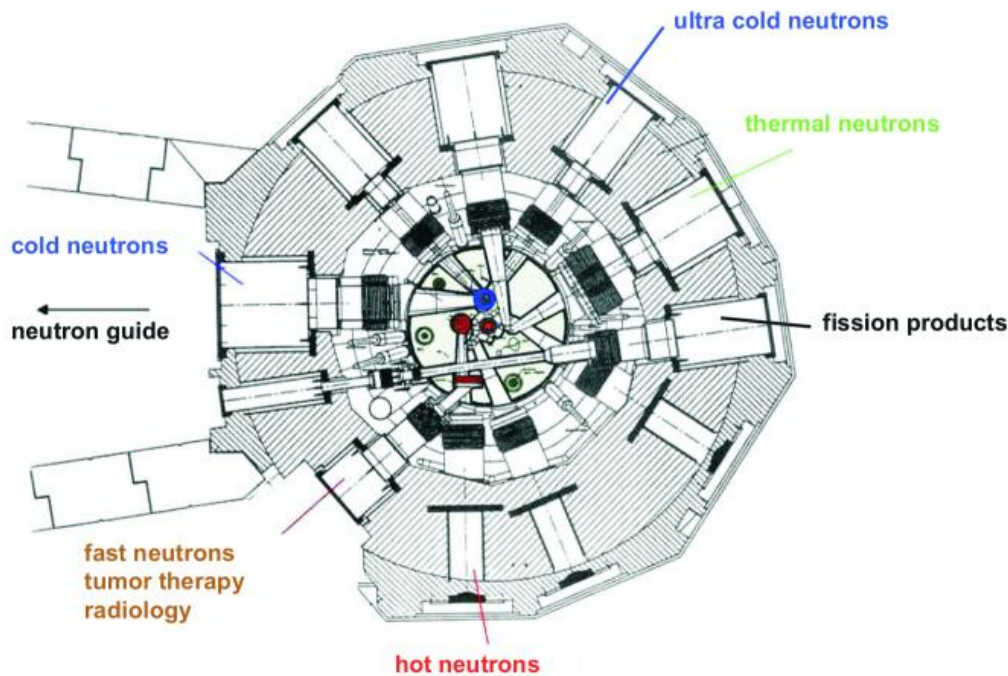
- E_n : from 0,1 MeV to 40 MeV
- Good energy resolution
- Reduced γ flash
- Low instantaneous flux

Average flux

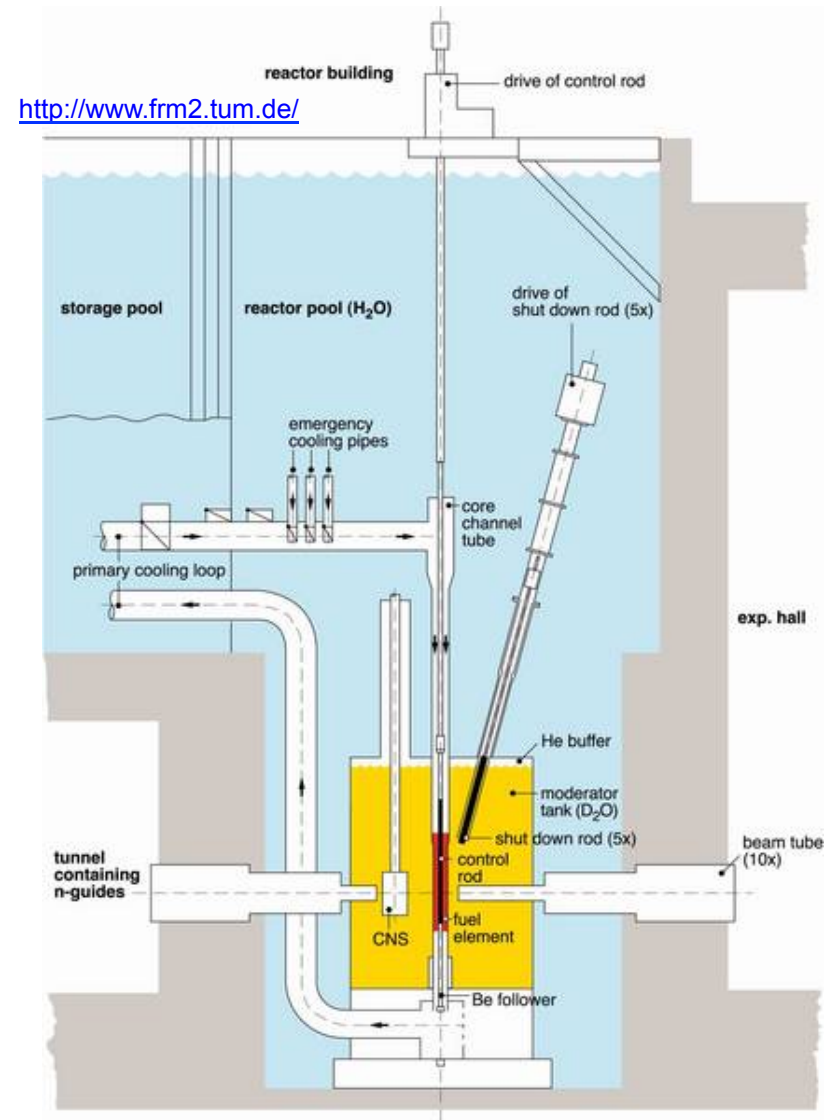


Complementary to the existing facilities

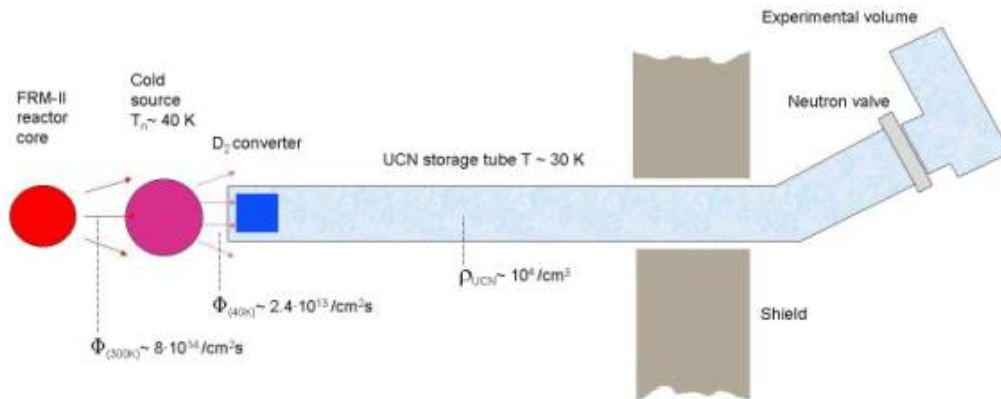
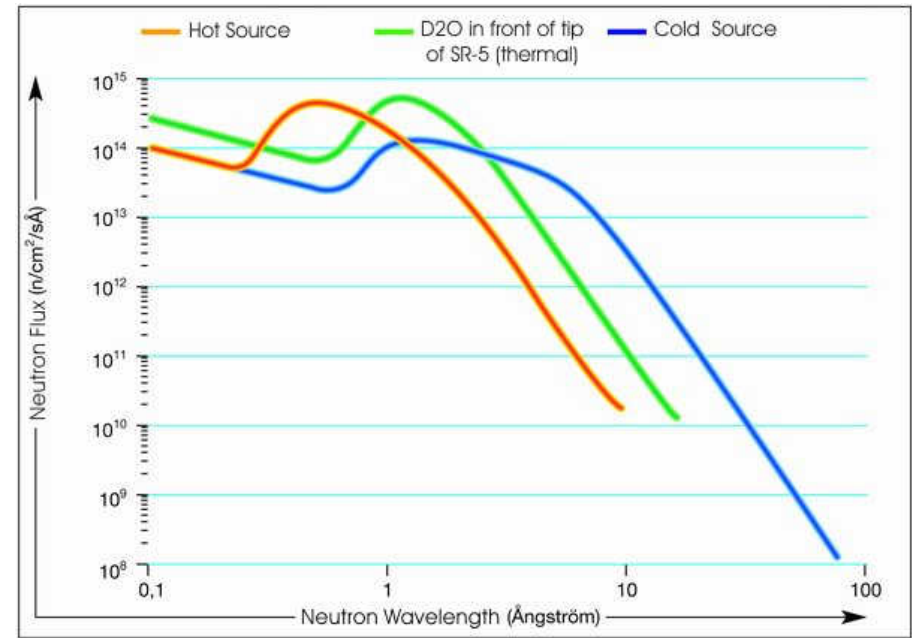
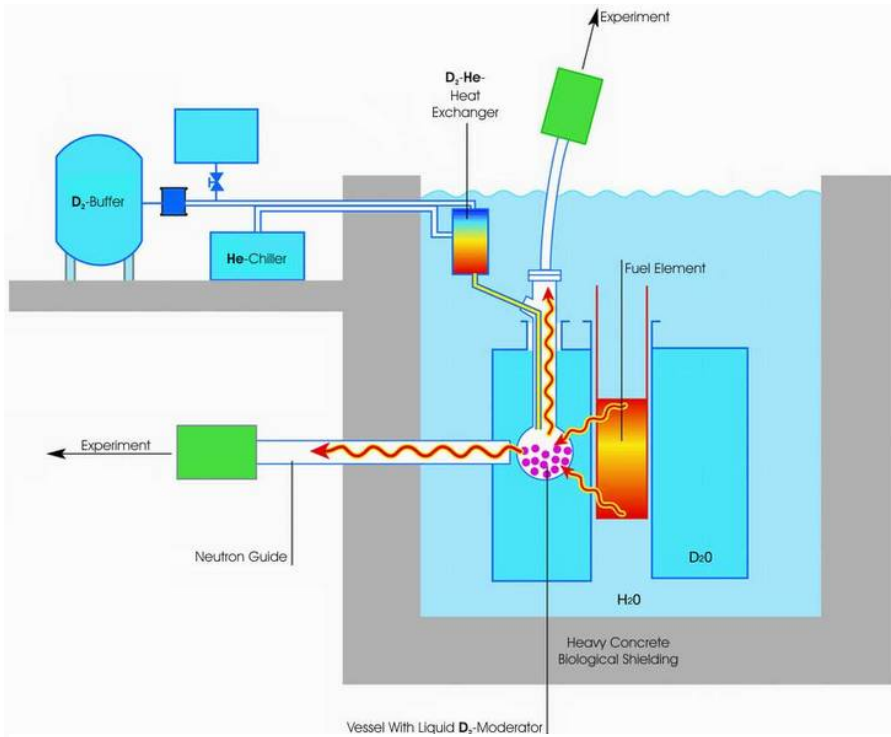
Research Reactor: Heinz Maier-Leibnitz FRM-II Garching



$P = 20 \text{ MW}_{\text{therm}}$
 1 fuel element 8 kg HEU (93% ^{235}U)
 neutron flux density: $8 \cdot 10^{14} \text{ n/(s cm}^2\text{)}$
 secondary sources:
 unmoderated fission spectrum $2 \cdot 10^9 \text{ n/(s cm}^2\text{)}$
Hot neutrons ($T = 2400 \text{ }^\circ\text{C}$)
cold neutrons ($T = 25 \text{ K}$) $9 \cdot 10^{13} \text{ n/(s cm}^2\text{)}$
Ultracold neutrons ($T = 5 \text{ K}$) $10^6 \text{ n/(s cm}^2\text{)}$



Neutron sources at FRM-II of TU Munich

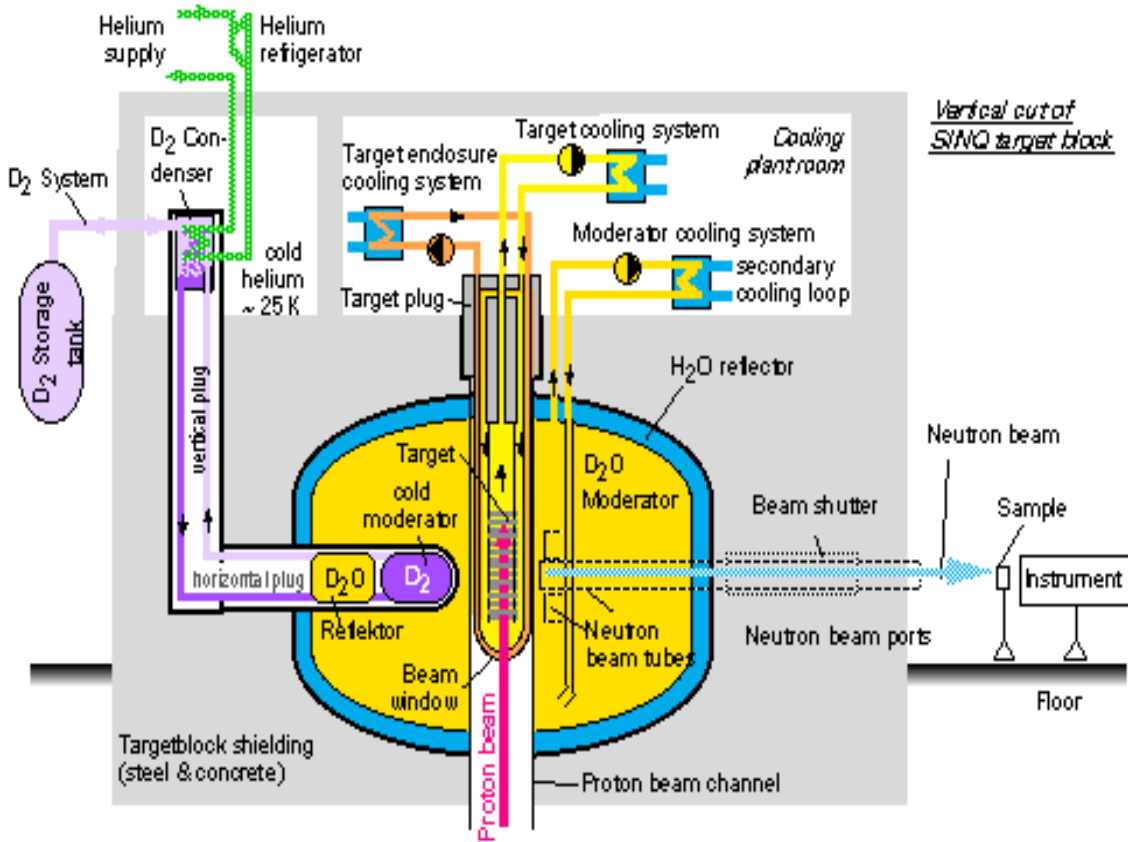


$$\frac{\lambda}{2\pi} = \frac{\hbar}{p} = \frac{\hbar}{\sqrt{2m_n E_{kin} + \frac{E_{kin}^2}{c^2}}}$$

$E_{kin} \text{ (eV)}$	$\lambda/2\pi$	
0.1	1.44 Å	crystal lattice constant
10^6	10 fm	nuclear diameter
10^9	1 fm	Nucleon diameter

Swiss Spallation neutron source [SINQ](#) Paul Scherrer Institute

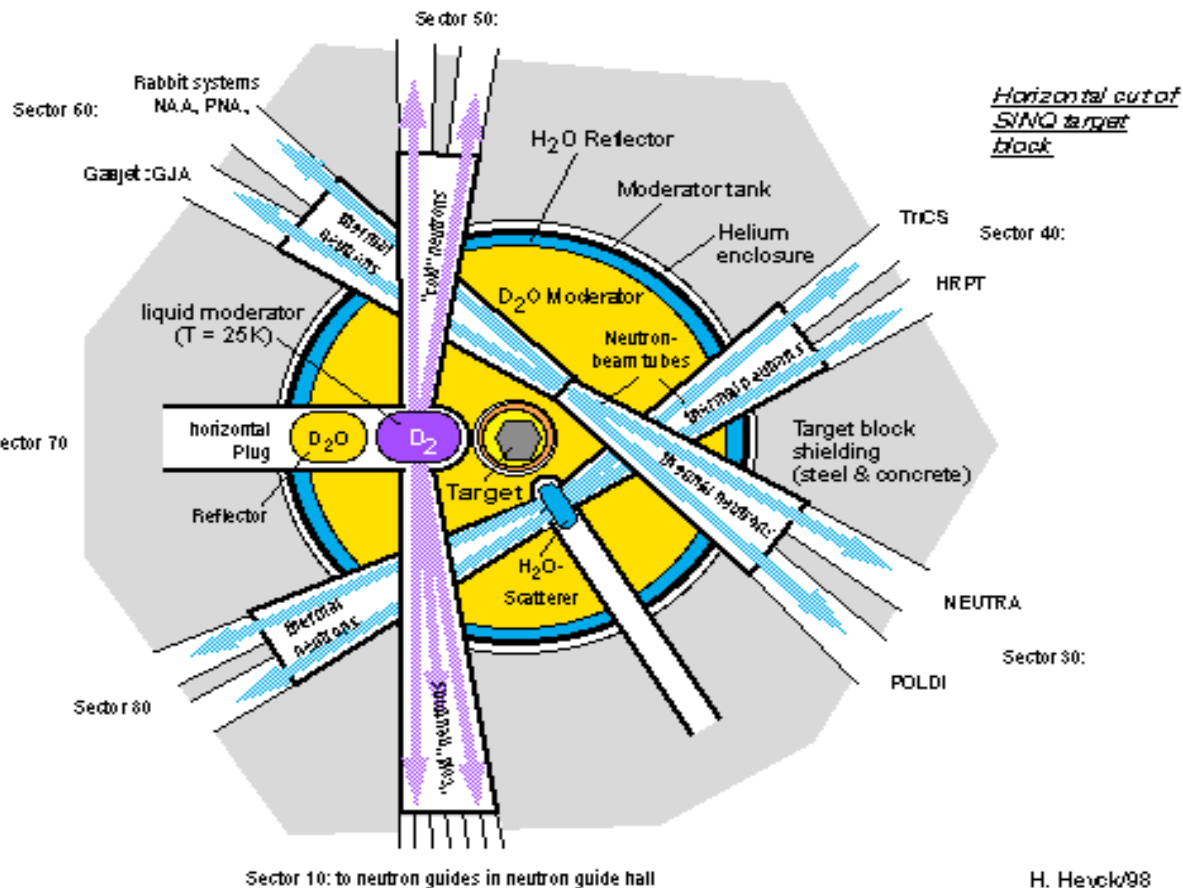
Principle of the Spallation Neutron Source SINQ



Proton beam 590 MeV, 1.8 mA
beam power ca. 1 MW
for pion production and
proton therapy

Pb spallation target
 $3\text{--}6 \cdot 10^{16}$ n/s
moderated and cold source
 10^{14} n/(cm² s)

Swiss Spallation neutron source SINQ Paul Scherrer Institute



ENDE

Nuclear Transmutation Project

- Roland Beyer, Evert Birgersson**, Anna Ferrari, Roland Hannaske, Mathias Kempe, Toni Kögler, Michele Marta, Ralf Massarczyk, Andrija Matic, Georg Schramm
- Arnd Junghans, Daniel Bemmerer, Eckart Grosse*, Klaus-Dieter Schilling, Ronald Schwengner, Andreas Wagner
- Development of the nELBE photoneutron source together with the Institute for Fluidynamics and the Central Research Technology Group

* (also at IKTP Dresden)

** now AREVA, Erlangen